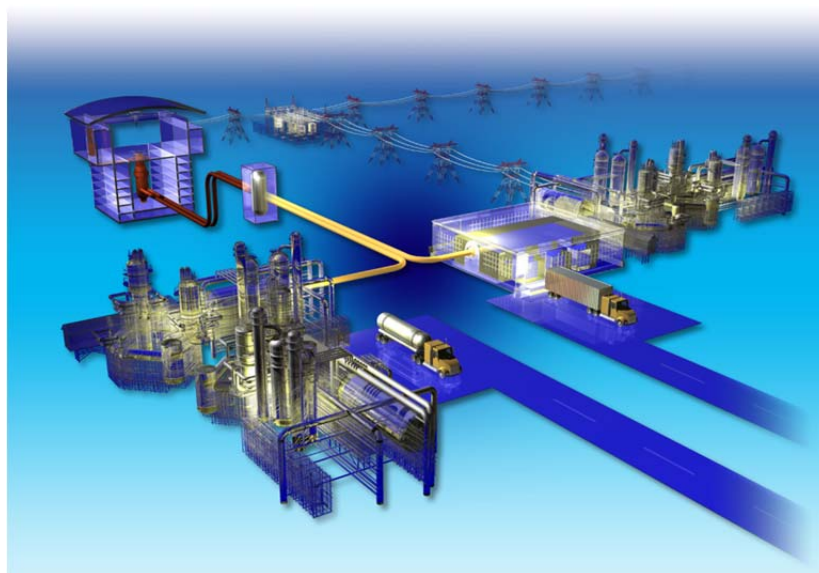


AGC-1 PIE Experimental Plan

Tim Burchell

Oak Ridge National Laboratory

March 2012



Prepared for
Office of Nuclear Energy Science and Technology
Prepared by
OAK RIDGE NATIONAL LABORATORY
Oak Ridge, Tennessee 37831
managed by
UT-BATTELLE, LLC
for the
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SUMMARY

Here we report the Post-Irradiation Examination planned for the AGC-1 creep samples. The key data obtained for each grade of graphite included in the AGC-1 capsule are:

- Dimensions, volume, mass and density changes
- Creep strain
- Electrical resistivity
- Elastic constants (E, G, ν) from sonic velocity
- Dynamic modulus (from fundamental frequency of vibration)
- Coefficient of thermal expansion
- Microstructural characterization
- Fracture strength

The effect of creep strain on the above properties will be determined.

These data are critical to the design of the Next Generation Nuclear Plant (NGNP). Moreover, the data supports ongoing work in the area of model development, e.g., irradiation effects models such as dimensional change and creep. The data will be used to help underpin the existing American Society of Mechanical Engineers (ASME) design code for graphite core components.

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ABBREVIATIONS

AGC	Advanced Graphite Creep
ASME	American Society of Mechanical Engineers
ASTM	American Society for Testing and Materials
ATR	Advanced Test Reactor
ER	Electrical Resistivity
INL	Idaho National Laboratory
LAMDA	Low Activation Materials Development and Analysis
MFC	Materials and Fuels Complex
ORNL	Oak Ridge National Laboratory
PIE	Post-Irradiation Examination
Pre-IE	Pre-Irradiation Examination

1 INTRODUCTION

The AGC-1 experiment is the first of a series of graphite creep capsules to be irradiated in the Advanced Test Reactor (ATR) at Idaho National Laboratory (INL) in support of the Next Generation Nuclear Plant (NGNP). Since the AGC-1 capsule was first conceived [1] it has undergone a number of design changes, requiring a revision of the original layout [2]. Here we describe the post irradiation examination (PIE) of the AGC-1 creep specimens to be conducted in the Low Activation Materials Development and Analysis (LAMDA) facility at the Oak Ridge National Laboratory (ORNL). This experimental plan describes the testing to be performed on the graphite creep specimens and SiC monitors only.

2 MATERIALS

A full description of the materials/specimens contained in this experiment may be found in the “AGC-1 Experimental Plan” [1] and in “A Revised AGC-1 Creep Capsule Layout” [2]. The graphite to be examined here are the AGC-1 creep specimens listed in Table 1. The pre-irradiation data [3] and sister specimens testing data [4] have previously been reported.

Table 1 Graphite grades and marking codes used for the AGC-1 series of creep specimens

Graphite Code Letter	Graphite Grade
A	NBG-17
B	NBG-18
C	H-451
D	PCEA
E	IG-110
F	IG-430

The graphite grades examined here all are in compliance with the ASTM specification for nuclear graphite [5].

3 MATERIAL POST-IRRADIATION EXAMINATION

Post-irradiation examination (PIE) will follow the workflow chart in Figure 1. Following irradiation in ATR the AGC-1 capsule will cool in the ponds followed by shipment to the hot cells. The capsule is disassembled in the hot cells at INL and the specimens and temperature monitors recovered and shipped to the Low Activation Material Development & Analysis (LAMDA) at ORNL. The specimens will undergo post irradiation examination (PIE) which will consist of visual inspection, dimensional measurements, creep strain, mass, and hence density, elastic constants (sonic velocity), dynamic Modulus by resonance frequency, electrical resistivity, and coefficient of thermal expansion. Additional microstructural characterization may be performed depending on schedule and budget. Fracture testing will also be conducted but the nature of testing (tensile, or compression) has not been determined at this time.

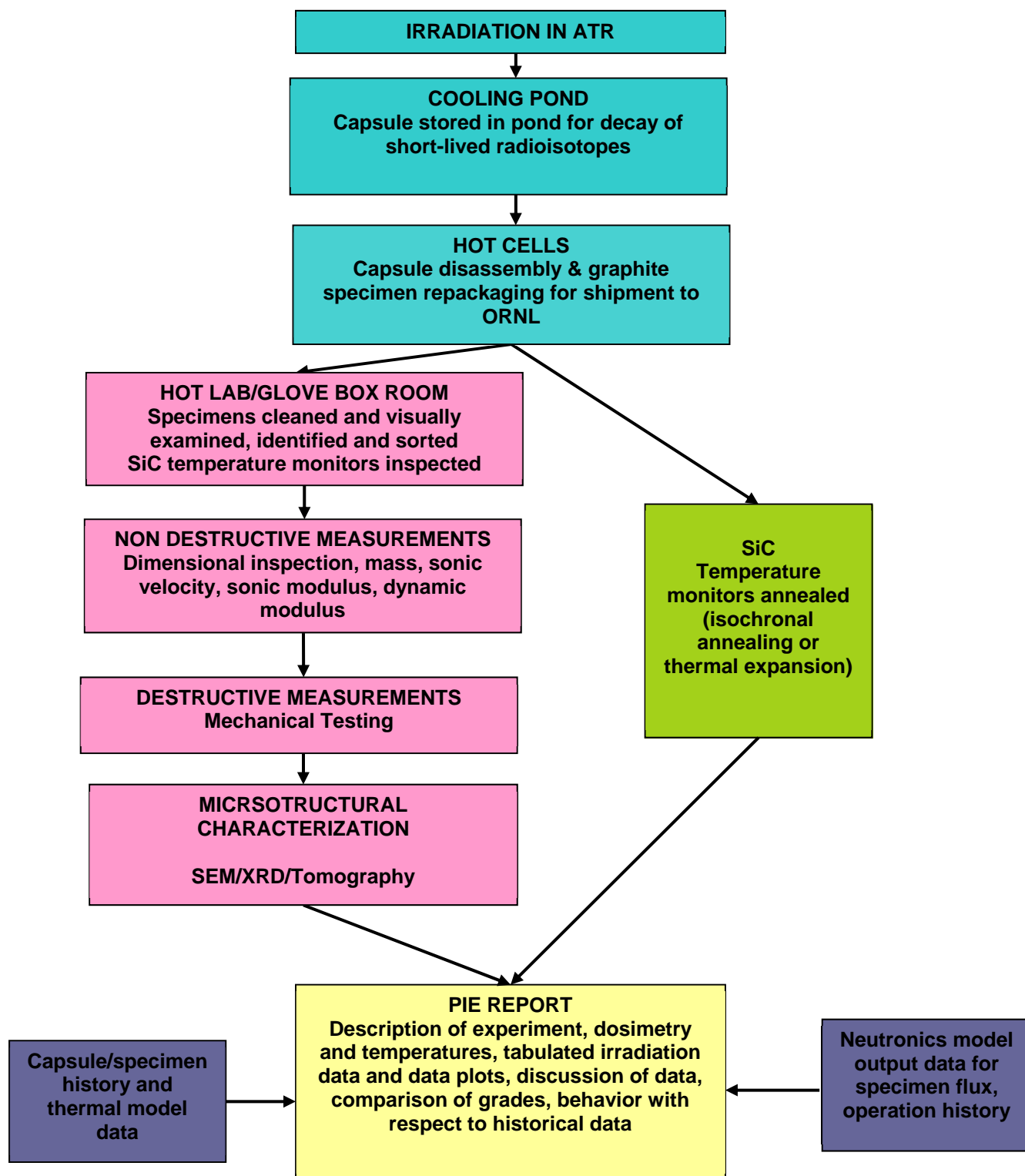


Figure 1 Post-irradiation data process flow chart for AGC-1 graphites

The specimen irradiation temperature will be determined from temperature monitors encapsulated within the specimens (Figure 2) in the AGC-1 central channel. The SiC monitors will be retrieved by INL and shipped to ORNL for testing. The irradiation temperature will be estimated by either (i) isochronal annealing followed by electrical resistivity (ER) determination using ASTM C611 [6], or (ii) thermal expansion. A plot of resistivity as a function of annealing temperature should display a marked change in resistivity when the annealing temperature exceeds the maximum irradiation temperature. The second method involves the continuous measurement of thermal expansion, in accordance with ASTM E228 [7] as the monitor is heated to a temperature in excess of the irradiation temperature. Again, the expansion data should exhibit a change in gradient when the irradiation temperature is exceeded.

The isochronal annealing temperature schedule is reported in Table 2.

Table 2 Annealing schedule for SiC temperature monitors

Annealing Temperature, °C	Heating Rate, °C/min	Time at Temperature, minutes	Actions
400	2.5	30	Cool to RT, Measure ER
450	2.5	30	Cool to RT, Measure ER
500	2.5	30	Cool to RT, Measure ER
550	2.5	30	Cool to RT, Measure ER
600	2.5	30	Cool to RT, Measure ER
625	2.5	30	Cool to RT, Measure ER
650	2.5	30	Cool to RT, Measure ER
675	2.5	30	Cool to RT, Measure ER
700	2.5	30	Cool to RT, Measure ER
725	2.5	30	Cool to RT, Measure ER
750	2.5	30	Cool to RT, Measure ER
775	2.5	30	Cool to RT, Measure ER
800	2.5	30	Cool to RT, Measure ER
850	2.5	30	Cool to RT, Measure ER
900	2.5	30	Cool to RT, Measure ER

Isochronal annealing furnace runs will be heated at 2.5 °C/minute, followed by a 30 minute dwell at temperature, cooled to room temperature and ER measured [6]. The experimental thermometry data will be contained in a final PIE report. Some of the SiC monitors were found broken on receipt from INL and thus were too short for ER measurement. These shorter monitors will be subjected to thermal expansion measurement. The SiC temperature monitors were heated at a rate of 2.5°C/minute to 1000°C while expansion was continuously measured according to E228 [7]. Table 3 indicates the SiC numbers, tube ID's, and the experimental measurements to be made.

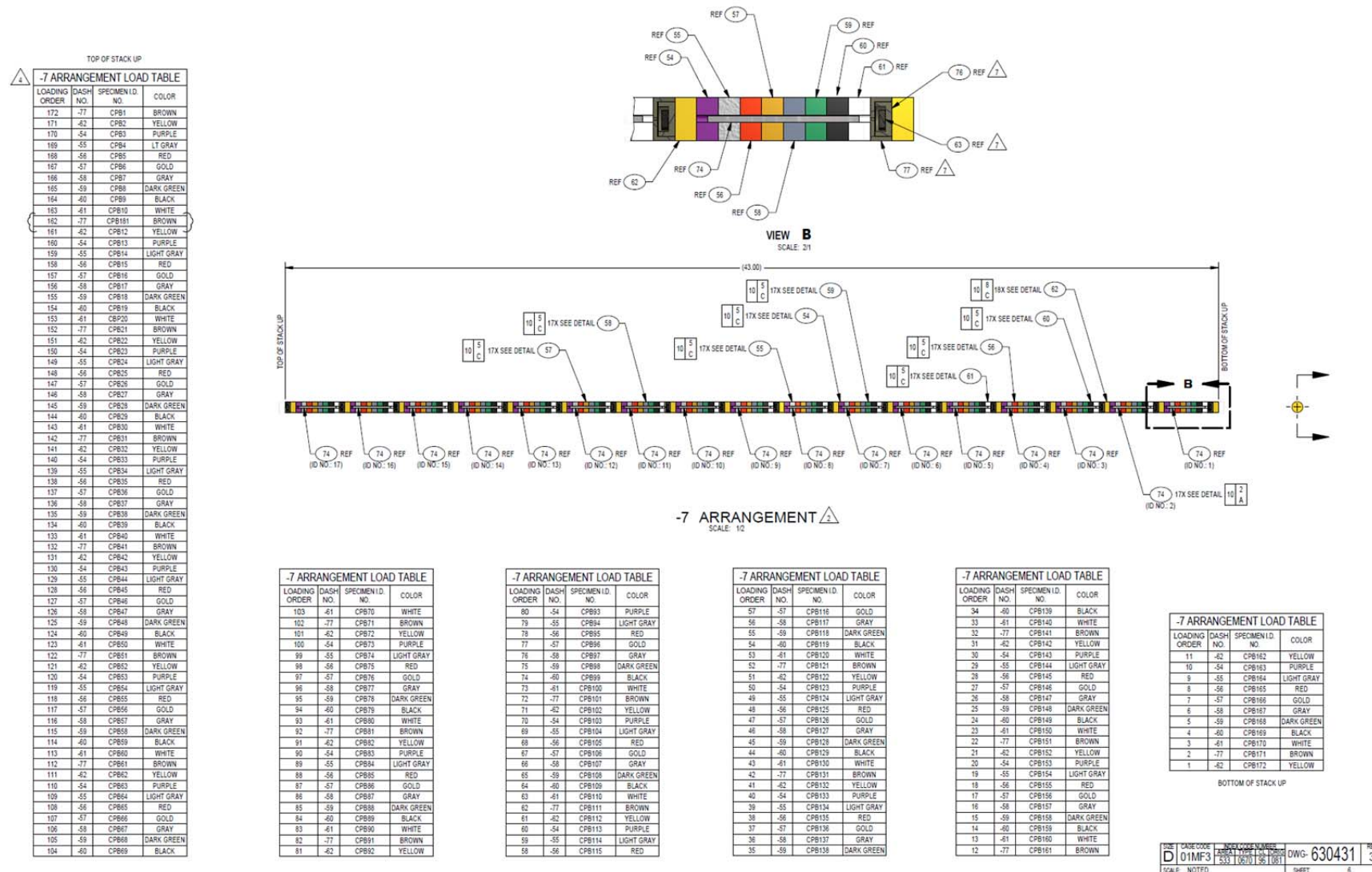


Figure 2 Drawing of AGC-1 capsule showing positions of SiC temperature monitors

Table 3 SiC temperature monitor information from AGC-1 (10-26-11) as supplied by INL

Number on Plastic Cryo Tube	Piggy back specimen associated with SiC TM in Plexiglass shipping tube from MFC.	SiC TM ID Number per INL Dwg. 630431 rev. 3.	Experimental Measurement Technique
1	CPB24	15	Elec. Resist.
2	CPB1-10	17	Elec. Resist.
3	CPB11-20	16	Elec. Resist.
4	CPB88	9	Therm. Expan.
5	CPB124, 125	5	Therm. Expan.
6	CPB76, 77, 79	10	Therm. Expan.
7	CPB156, 157	2	Elec. Resist.
8	CPB118, 119	6	Elec. Resist.
9	CPB167, 168	1	Elec. Resist.
10	CPB145, 144	3	Elec. Resist.
11	CPB110, 109	7	Elec. Resist.
12	CPB98, 99	8	Elec. Resist.
13	CPB68, 70	11	Elec. Resist.
14	CPB55, 57, 60	12	Elec. Resist.
15	CPB49, 47	13	Elec. Resist.
17	Packaged alone in bottle. By process of elimination this is either ID no. 14 or 4 and therefore no. 14 or 4 are missing.	14 or 4	Elec. Resist.

4 EXPERIMENTAL PIE METHODS

The applicable ASTM test standards used here are listed in Table 4.

Table 4 Applicable ASTM test methods

Test Property	Standard Title	ASTM Standard	Ref No.
General	Testing Graphite and Boronated Graphite Materials for High-Temperature Gas-Cooled Nuclear Reactor Components	C781	[8]
Dimensions, mass, density	Bulk Density by Physical Measurements of Manufactured Carbon and Graphite Articles	C559	[9]
Electrical Resistivity	Electrical Resistivity of Manufactured Carbon and Graphite Articles at Room Temperature	C611	[6]
Dynamic Young's Modulus	Moduli of Elasticity and Fundamental Frequencies of Carbon and Graphite Materials by Sonic Resonance	C747	[10]
Dynamic Young's Modulus	Dynamic Young's Modulus, Shear Modulus, and Poisson's Ratio for Advance Ceramics by Impulse Excitation of Vibration	C1259	[11]
Yong's Modulus	Sonic Velocity in Manufactured Carbon and Graphite Materials for Use in Obtaining Young's Modulus	C769	[12]
Thermal Expansion	Linear Thermal Expansion of Thermal Materials with a Push-Rod Dilatometer	E228	[7]
Compressive Strength	Compressive Strength of Carbon and Graphite	C695	[13]
Tensile Strength	Tensile Stress-Strain of Carbon and Graphite	C749	[14]
Glued End Tensile Strength	Testing Graphite and Boronated Graphite Materials for High-Temperature Gas-Cooled Nuclear Reactor Components	C781 A4 (mod to C749)	[15]

The order of PIE testing shall be:

1. Visual inspection, photography
2. Dimensions, mass, density
3. Electrical resistivity
4. Dynamic Young's modulus by resonance
5. Sonic velocity and elastic constants
6. Thermal expansion, CTE
7. Strength
8. XRD, SANS, tomography, microscopy

Dimensional measurements will be taken according to the scheme in Figure 3 so as to assure complete repeatability from the pre-irradiation measurements. Similarly, specimen alignment (with respect to the engraved number) during PIE physical property measurements shall be the

same as previously employed during Pre-IE [3]. Any testing limitations in regard of specimen size shall be in accordance with C781[8].

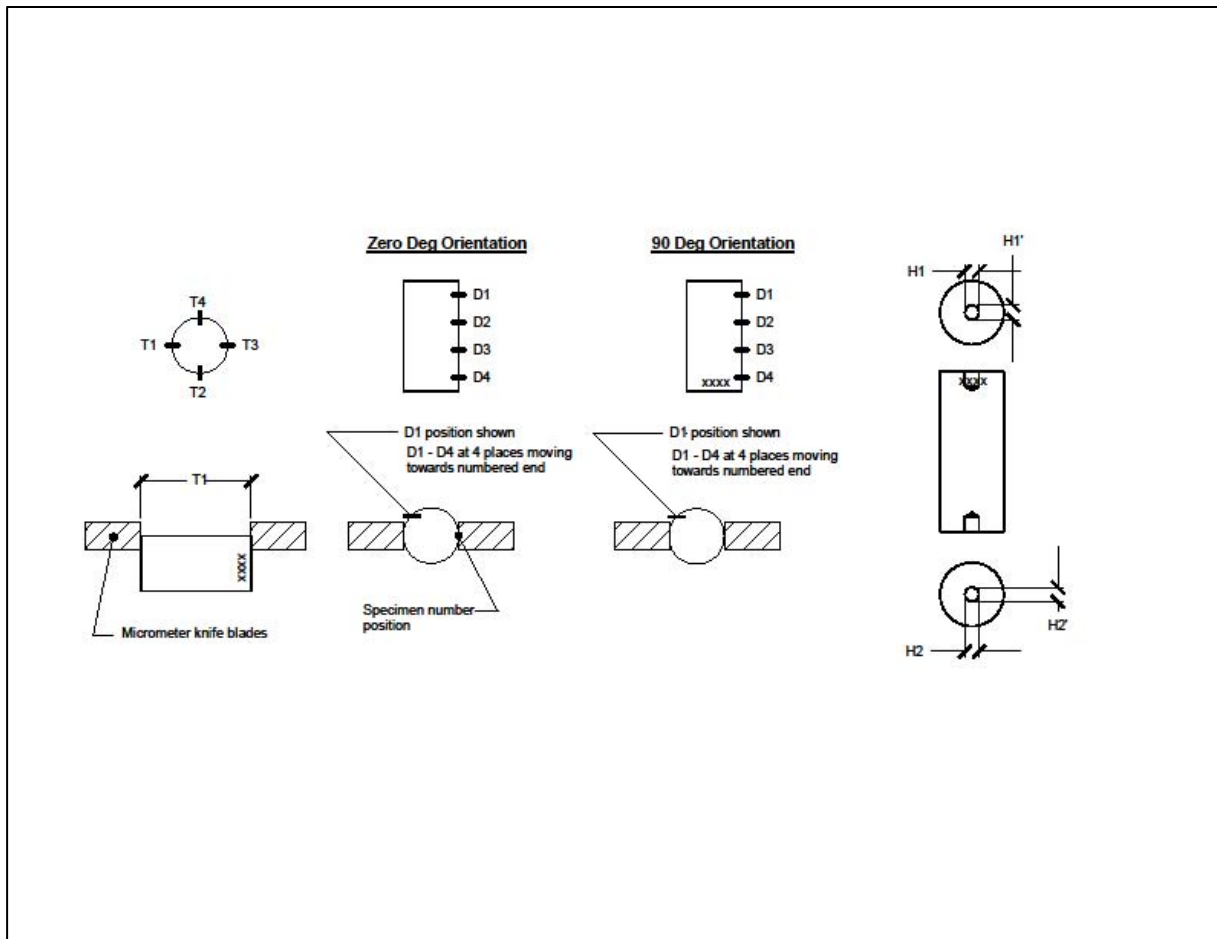


Figure 3 AGC-1 creep specimen measurement scheme

5 QUALITY ASSURANCE

The activities described in this plan will be conducted in accordance with the applicable requirements of the ASME/NQA-1-2000 national standard entitled *Quality Assurance Requirements for Nuclear Facility Applications*. Project and activity-specific information concerning ORNL's application of the standard's requirements is provided in Document #QAP-ORNL-NGNP-01 entitled [Quality Assurance Plan for the Next Generation Nuclear Plant Materials Program at Oak Ridge National Laboratory](#) [15].

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