

Test Report of Special Form Qualification Testing for the ORNL U ZIPCAN



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August 2017

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Reactor and Nuclear Systems Division

**TEST REPORT OF SPECIAL FORM QUALIFICATION
TESTING FOR THE ORNL U ZIPCAN**

O. A. Martinez, Ph.D.

August 2017

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US DEPARTMENT OF ENERGY
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ACRONYMS

ANSI	American National Standards Institute
ASTM	American Society for Testing and Materials
CFR	Code of Federal Regulations
NDT	non-destructive testing
NSC	Y-12 National Security Complex
ORNL	Oak Ridge National Laboratory
PTP	Package Testing Program
QA	quality assurance
REDC	Radiochemical Engineering Development Center
RHAC	Research Hazard Assessment and Control
TIG	tungsten inert gas
TU	test unit
ZiPCans	Zirconia Pre-Encapsulation Canisters

ABSTRACT

Two prototype Zirconia Pre-Encapsulation Canisters (ZiPCans) of the same design were evaluated to demonstrate compliance with requirements of the following regulations:

- Title 49, Code of Federal Regulations (CFR), Part 173.469, *Tests for Special Form Class 7 (Radioactive) Materials*, and
- Title 10, Code of Federal Regulations, Part 71.75 (1)(i), Qualification of special form radioactive material and ISO2919:1999(E) Radiological protection –Sealed radioactive sources – General requirements and classification.

The results of the special form tests are documented in this test report.

This test report describes the special form testing activities performed on the two ZiPCans. One prototype test unit was subjected to the tests stipulated by 10 CFR 71.75 (d)(1)(i), ISO 2919:1999(E) Class 4 impact test, along with the leak rate test specified in 49 CFR 173.469(a)(4)(i). The other test unit was subjected to a leak rate test as specified in 173.469(a)(4)(i) and a heat test as specified in 49 CFR 173.469 (b)(4). Each test unit was leak tested before and after these respective tests. The leak rate tests performed were helium back-pressure tests and bubble tests, as specified in ANSI N14.5-2014. The measured leak rates were converted to standard condition leak rates as specified in ASTM E 493. The determined standardized leak rates from the test and calculation for both test units met the requirements for special form certification.

The testing was performed by or under the direction of the Oak Ridge National Laboratory (ORNL) Package Testing Program (PTP).

1. INTRODUCTION

Two prototype ZiPCans designed to contain uranium (henceforth referred to as *U ZiPCans*), were tested to demonstrate compliance with the requirements of

- Title 49, Code of Federal Regulations (CFR), Part 173.469, *Tests for Special Form Class 7 (Radioactive) Materials*, and
- Title 10, Code of Federal Regulations, Part 71.75, Qualification of special form radioactive material and ISO2919:1999(E) Radiation protection –Sealed radioactive sources – General requirements and classification.

These prototypes served as test units and are identified as TU-1 (C1-0290), and TU-4 (OPSF1).

The 10 CFR 71.75 requirement states:

(d) A specimen that comprises or simulates radioactive material contained in a sealed capsule need not be subjected to —

- (1) The impact test and the percussion test of this section, provided that the specimen is:
(i) Less than 200 grams and alternatively subjected to the Class 4 impact test prescribed in ISO 2919:1999(E) “Radiation protection –Sealed radioactive sources – General requirements and classification”*

Since the ZipCan design is less than 200 g, the TU-4 U ZiPCan was subjected to a Class 4 impact test only as prescribed in ISO2919:1999(E) in lieu of the percussion and impact test described in 49 CFR 176.469 (b).

The ISO2919:1999(E) impact test is specified below:

7.4 Impact Test

7.4.1 Apparatus

7.4.1.1 Steel hammer, the upper part of which is equipped with a means of attachment, and the lower part of which shall have an external diameter of (25 ± 1) mm and a flat striking surface with its outer edge rounded to a radius of (3.0 ± 0.3) mm.

The center of gravity of the hammer shall lie on the axis of the circle, which defines the striking surface; this axis itself passing through the point of attachment. The mass of the hammer for each test class is given in Table 2.

7.4.1.2 Steel anvil, the mass of which is at least ten times that of the hammer. It shall be rigidly mounted so that it does not deflect during impact and shall have a flat surface, large enough to support the entire sealed source.

According to Table 2 from ISO 2919:1999(E), the weight of the steel hammer for the Class 4 impact test shall be “2 kg from 1 m or equivalent imparted energy.” Based on the equation of potential energy to total imparted energy, the imparted energy shall be 19.61 Joules = $2 \text{ kg} \times 9.81 \text{ m/s}^2 \times 1 \text{ m}$. Additionally, TU-4 subsequently was subjected to a leak rate test before and after each of the tests described above to determine test outcome, as follows:

Leak Rate Test (49 CFR 173.469 (a)(4)(i))

Demonstration of leak tightness of 10^{-4} torr-1/s (3.1×10^{-4} atm-cm³/s) based on air at 25°C (77°F) and one atmosphere differential for solid radioactive content .

..

TU-1 underwent one heat stress test, as well as leak rate testing, as described above, before and after the heat stress test:

Heat Stress Test (49 CFR 173.469 (b)(4)):

The specimen must be heated in air to a temperature of not less than 800°C (1475°F), held at that temperature for a period of 10 minutes, and then allowed to cool.

All tests (impact test, heat stress tests, and leak rate tests) were performed by or under the direction of the Oak Ridge National Laboratory (ORNL) Package Testing Program (PTP). This report provides a detailed description of the test methodologies and results.

1.1 DESCRIPTION OF THE U ZIPCAN

The inner container of the U ZiPCan is a titanium triangular assembly with four threaded $3/32$ -inch fill holes over four inner triangular cavities matted with a 0.05-inch thick zirconium oxide felt. Uranium heat stress test unit (TU-1) was manufactured by depositing drops of a nitrate solution into the inner triangular

cavities through the fill holes. After the solution was deposited, the filled triangular assembly was slowly heated to concentrate the liquid to a dry salt and then was subsequently fired in a furnace to convert the uranium material to an oxide. Four titanium screws were then inserted into the threaded fill holes. The assembly was then placed in a stainless steel triangular encasement, and fitted with a lid which had been welded with a tungsten inert gas (TIG) arc welder. The U ZiPCan is shown in parts in Figure 1.1 and Figure 1.2, and the impact test unit is shown in Figure 1.3.

A loaded U ZiPCan containing a maximum of 3.2 g (element weight) of uranium oxide was used for the heat test, and an unloaded ZiPCan was used for the impact test. The isotopic distribution of TU-1 is shown below.

Table 1.1. Isotopic distribution of the heat test unit, TU-1 (C1-0290)

ORNL U ZiPCan load information				
Tile/serial No.	C1-0290		Weight, g	Weight, fraction
Total weight	35.783 g	²³⁴ U	3.146E-05	9.831E-06
UO₃/U₃O₈ weight	4.58 g	²³⁵ U	1.295E-03	4.048E-04
Uranium weight	3.2 g	²³⁶ U	3.173E-05	9.916E-06
Isotopic mass date	3/23/2017	²³⁸ U	3.199E+00	9.996E-01

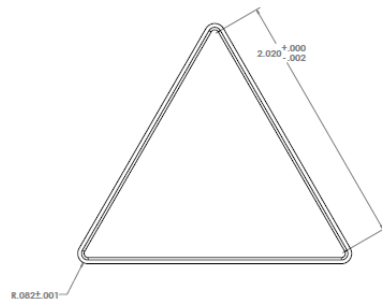


Figure 1.1. Top view of the U ZiPCan triangle encasement.

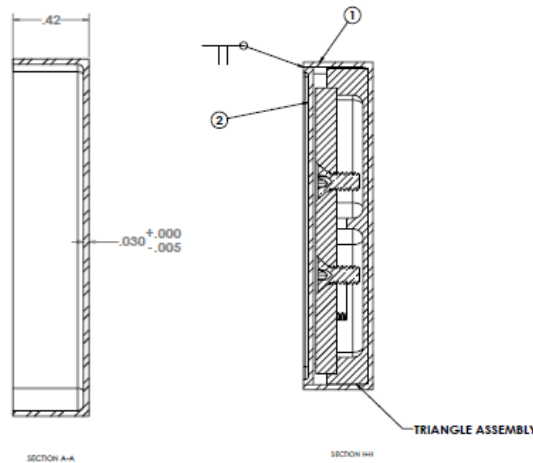


Figure 1.2. Side assembly view.

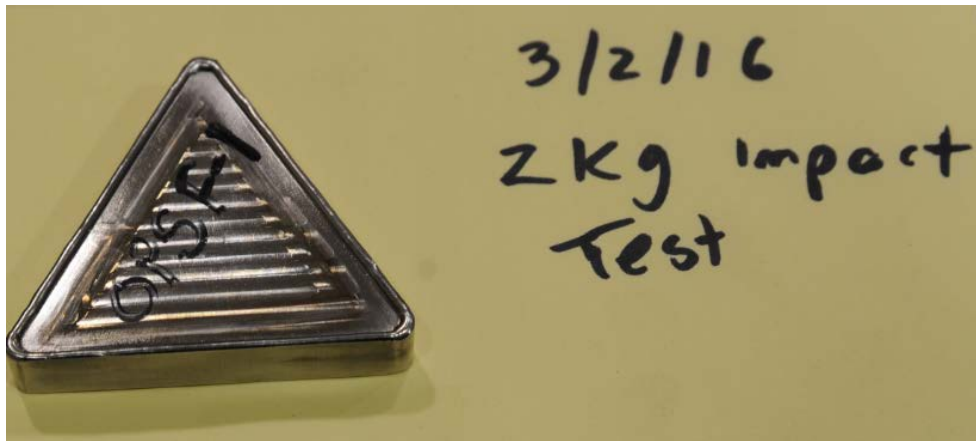


Figure 1.3. U ZiPCan triangle encasement test unit.

1.2 DESCRIPTION OF QUALITY ASSURANCE ACTIVITIES

All DOE contractors are required by contract with the US Government to comply with DOE Order 414.1D, 10 CFR 830.120 and/or other specific quality assurance (QA) requirements. Specific QA programs apply to each of three primary phases of effort (design, manufacture, and certification testing). ORNL was responsible for the design process, and the applicable QA program is the ORNL Quality Management System, *Quality Assurance Program Description*. The Radiochemical Engineering Development Center at ORNL was responsible for all manufacturing activities, and the two applicable QA programs are the Quality Management System described in NMP-QM-1, Rev. 1, *Quality Manual for the Nuclear Material Processing Group*, and the Nonreactor Nuclear Facility Division NNFD-017-C, Rev. 1, *NNFD Fabrication Control Procedure*.

For the certification testing process, each test was conducted in accordance with ORNL/NTRC-074 entitled *Test Plan for the Special Form Qualification Testing of the U ZiPCan Triangle Encasement* (available upon request) and the appropriate procedures listed in the test plan. The QA aspects of activities in the test plan are controlled by the PTP QA requirements of 10 CFR 830.122. The safety aspects of activities in this test plan are controlled by the ORNL Research Hazard Assessment and Control (RHAC) Research Safety Summary (RSS) 1082, titled *General Use and Package Testing Activities Conducted in the NTRC Packaging Research Facility*. Additionally, all testing performed by PTP is conducted under the QA plan outlined in NTRC-PRF-QAP-001, Rev. 2, *Quality Assurance Plan for the Package Testing Program*.

1.3 ZIPCAN TEST MATRIX

TU-1 is a U ZiPCan loaded with 3.2 g of uranium Table 1.2 provides the sequence of the tests and processes performed on each test unit. The number in the cell indicates the sequence in which the process or test was performed on the test unit. TU-4 is a U ZiPCan without radioactive material. It was deemed that the radioactive material provided a negligible amount of support to the titanium structure. The weight added is also irrelevant due to substitution of the ISO class for impact test as applicable to the drop test.

Table 1.2. Sequence of Tests and Processes for the U ZiPCan

Test or process description	Test unit	
	TU-1 (C1-0290)	TU-4 (OPSF1)
Leak test	1	1
Impact test (ISO 2919)	-	2
Heat test	2	-
Leak test	3	3

1.4 TEST DATA RECORDS

This report documents the tests performed and measurements observed from the U ZiPCan testing. The general data types for these tests are:

- manually derived measurements and observations,
- digital still photography, and
- video recording of the drop and percussion tests.

The primary recording media for each of the general types of data are:

- procedure checklists, data sheets and test forms for data, measurements, and observations,
- computer files (JPG format) of the digital photography, and
- computer files (MPG format) of the video recordings.

The completed data sheets and procedure checklists have been scanned into a digital format and are available upon request. Photographs are presented in the main body of this document as appropriate.

1.5 DEVIATIONS FROM THE TEST PLAN

Per the test plan (ONRL/NTRC-074), 3 test units were subjected to the preheat leak test and a heat test.

Table 1.3 provides a detailed sequence for the tests conducted on Test Units TU-1, TU-2 and TU-3.

Table 1.3. Detailed sequence of tests and processes for Test Units 1–3 (TU-1 through TU-3)

Test Unit TU-1, TU-2, TU-3 Sealed encapsulated specimen with radioactive material (U)	Acceptance criteria
Test sequence #1: Leaktightness	Leak Test - 49 CFR 173.469 (a)(4)(i)^{a,b,c} Leak pretest to ensure that there is no leakage prior to performance of heat test.
Test sequence #2: Heat test	Heat Test - 49 CFR 173.469 (b)(4): The specimen may not melt or disperse when subjected to the heat test Reference 49 CFR 173.469 (a)(3) ^a
Test sequence #3: Leaktightness	Leak Test - 49 CFR 173.469 (a)(4)(i)^{a,b,c} Leakage post-test to ensure that there is no leakage after heat test.

^a After each test, leaktightness of the specimen must be determined. Reference 49 CFR 173.469(a)(4).

^b Perform test for leaktightness per 49 CFR 173.469 (a) (4) (i). NOTE: The test specimen capsule must be fabricated from corrosion-resistant material that is resistant to corrosion by water and must have an internal void volume greater than 0.1 millimeters. Leaktightness testing acceptance criteria must demonstrate a leak tightness of 10⁻⁴ torr-1/s (1.3 × 10⁻⁴ atm-cm³/s) based on air at 25°C (77°F) and one atmosphere differential pressure for solid radioactive content. This test method is more sensitive than the leaching assessment methods specified by 49CFR 173.469 (4)(c).

^c Leaching assessment methods for indispersible solid material do not apply. Reference 49 CFR 173.469 (4)(i).

Table 1.4 shows data for the test units that were subjected to the heat test with the corresponding weigh of radioactive material. The post leak test was first performed on the heavy test unit (C1-0290), and that test unit passed the leak test; therefore, leak tests were not performed for the remaining heat test units because the leak test results for the 3.2 g test unit will cover the 2.5 g and 1.7 g test unit. This test report only reports the leak test results and heat test results of the 3.2 g test unit (C1-0290).

Table 1.4. Heat Test Units mass distribution

Test unit SN	Test unit number	Uranium weight (g)
C1-0290	1	3.2
C1-0288	2	2.5
C1-0289	3	1.7

2. PRE-TEST ACTIVITIES

The test units were delivered for testing in a ready-to-test condition, so there were no specific pretest activities.

3. SPECIAL FORM TESTS

Special form testing requirements are specified in 49 CFR 173.469 (b), 10 CFR 71.75, and ISO 2919:1999(E). For this design, three tests were required: the Class 4 impact test per ISO 2919:1999(E), a heat test, and a leak test. The bending test was not required because the length-to-width ratio of the design is not greater than 10. After each test, each test unit was subjected to a helium leakage rate test and a bubble test as specified in 49 CFR 173.469(a). Each test performed and the results of these tests are described below.

3.1 IMPACT TEST (ISO 2919:1999(E))

Test unit 4, TU-4 (OPSF1) was subjected to the ISO 2919:1999(E) Class 4 impact test, which was carried out at the indoor drop pad located at the National Transportation Research Center in Knoxville, Tennessee. This drop pad has a total mass of ~13.6 metric tons and meets the specifications for the impact test target according to the *Design and Certification of Targets for Drop Testing at the NTRC Package Research Facility Rev. 0*, May 2003, ORNL/NTRC-001. The test was performed according to the procedure outlined in the *Test Plan for the Special Form Qualification Testing of the U ZiPCan Triangle Encasement*, ORNL/NTRC-074, Section 3.7.1. Testing activities and results were recorded on Test Form 1 from the test plan.

The TU-4 U ZiPCan was centered on the indoor drop pad. A 1-inch diameter steel billet, weighing 2 kg was placed on the release mechanism and raised to a height of 1 meter. A calibrated meter stick was used to measure the height from the bottom surface of the billet to the top surface of the U ZiPCan (Figure 3.1). When ready, the steel billet was released so that the billet made a direct impact on the vertex of TU-4.

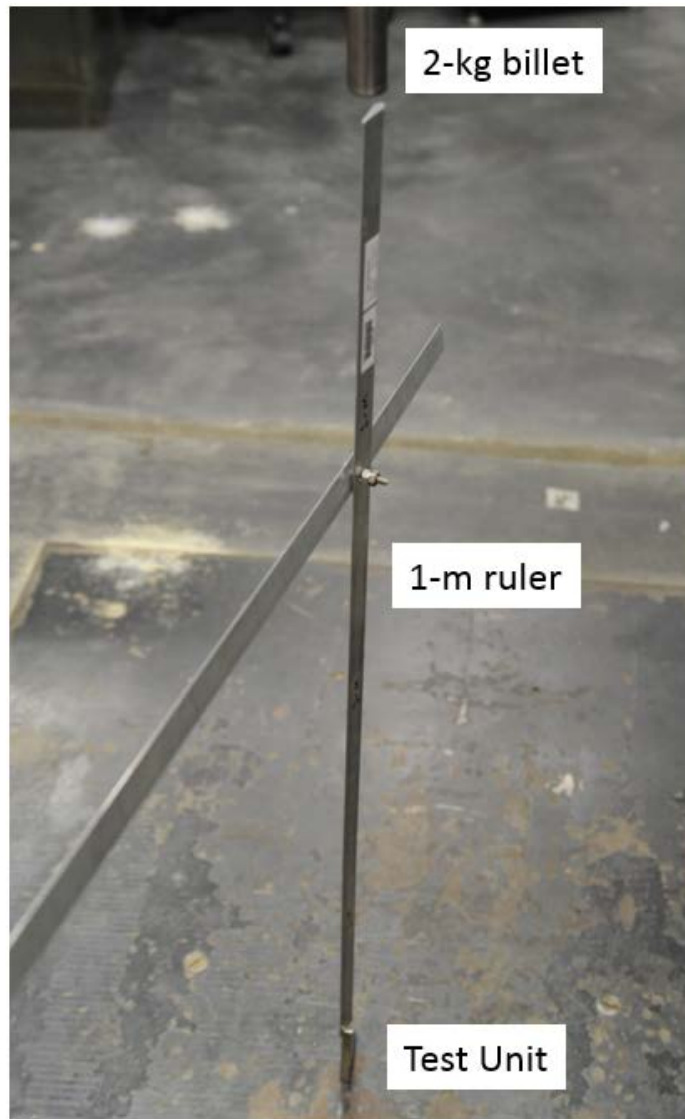


Figure 3.1. Impact billet 1 m above the ZiPCan.

When released, the billet appeared to impact TU-4 squarely on the vertex. The impact of the billet resulted in a slight indentation at the point of impact. Figure 3.2 photos show TU-4 before and after the impact test. After the impact test, TU-4 was subjected to a fine and gross leak test as described in Section 3.3.

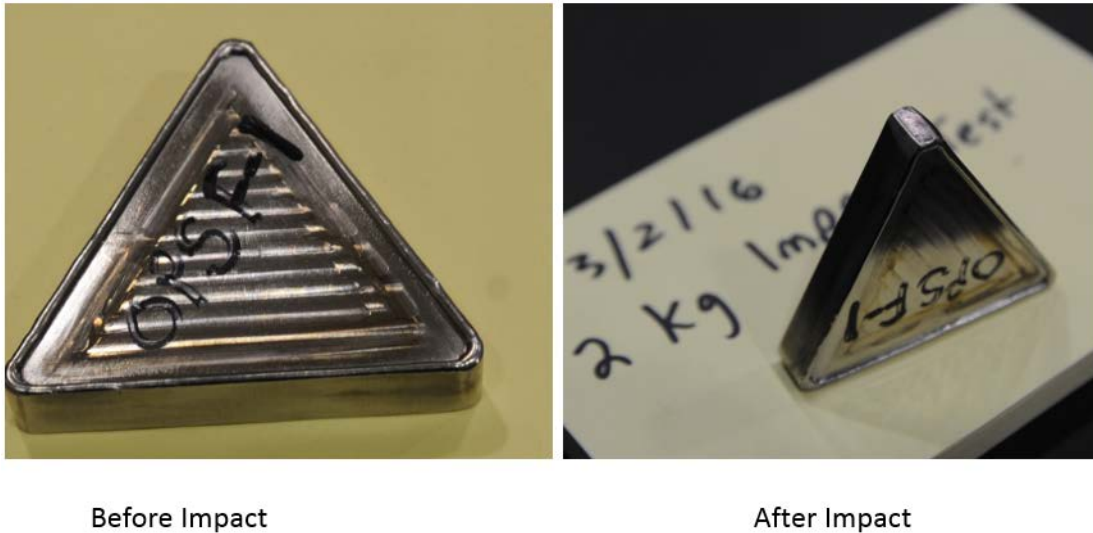


Figure 3.2. Before and after impact of the U ZiPCan.

3.2 HEAT TEST

The 49 CFR 173.469(b)(4) heat test was performed on the TU-1 (C1-0290) test unit which had been loaded with 3.2 grams of depleted uranium. The special form tile loading log can be found in ORNL Log Book H00034-RSTD Fabrication. The uranium was depleted in the Y-12 National Security Complex (NSC) cauldrons to a high percentage of ^{238}U as batch number D7. The isotopes for the batch D7 uranium can be found in Table 1.1. The test was conducted in the ORNL Radiochemical Engineering Development Center (REDC), Building 7930, Lab 212 Fume Hood (IE-960). The safety aspects of activities for this heat test are controlled by the ORNL Research Hazard Assessment and Control (RHAC) Research Safety Summary (RSS) 919, *REDC Bldg. 7930 Development Laboratory Operations*. The furnace used was a Thermolyn Model #F47925, Serial No. 0152853201110405, property number 18334 (Figure 3.). The furnace has a noncalibrated integrated controller. Two 12-inch Type K thermocouple probes (BF3874 and BF3F05) were calibrated before the test and inserted into the top of the furnace and extended into the center of the furnace cavity (Thermocouple 1 - BF3874; Thermocouple 2 - BF3F05 Figure 3.4). The probe was connected to a calibrated fluke thermometer B1332, Serial No. 36370410WS, with a calibration due date of 9/27/2017.



Figure 3.3. Heat test furnace in REDC.

Job# 3054371						Tech: 30220
Date: 1/24/17		Technical Support Department				Std: A001277
		Instrument Data Continuation Sheet				M210101
						A002021
Furnace	Standard	UUT Reading				
	Type S	BF3874	Error	BF3F05	Error	
21.5	21.5	20.9	-0.6	21.1	-0.4	
750.0	754.1	755.0	0.9	754	-0.1	
800.0	803.5	804.5	1.0	803.6	0.1	
850.0	854.0	855.0	1.0	854.4	0.4	
900.0	904.3	905.4	1.1	905.1	0.8	
950.0	955.4	956.9	1.5	956.3	0.9	

Thermocouple 1 - BF3874; Thermocouple 2 - BF3F05

Figure 3.4. Type K thermocouple calibration record.

The test was performed according to the procedure outlined in the *Test Plan for the Special Form Qualification Testing of the U ZiPCan Triangle Encasement*, ORNL/NTRC-074, Section 3.7.2, and testing activities and results were recorded on Test Forms 2, 3, and 4 from the test plan. The furnace was preheated above 800°C for three hours. After a three-hour heat soaking period at a constant temperature of 980°C, the furnace door was opened, and TU-1 was inserted into the furnace cavity. The furnace door was closed, and when both thermocouples had a furnace reading above 800°C, the 10-minute thermal test was started (Figure 3.5). Thermocouple 1 is BF3874, and thermocouple 2 is BF3F05. A noncalibrated

stop watch was used to record the temperatures from both thermocouples every 30 seconds for 10 minutes. After the 10-minute period, the door was opened, and TU-1 was removed from the furnace and allowed to cool naturally. The thermal test resulted in an out-of-plane bulge (pillow effect) of the test unit, which is shown in Figure 3.6. There was a discoloration on the outer surface of the test unit. The test unit was helium leak tested and bubble tested after the thermal test.

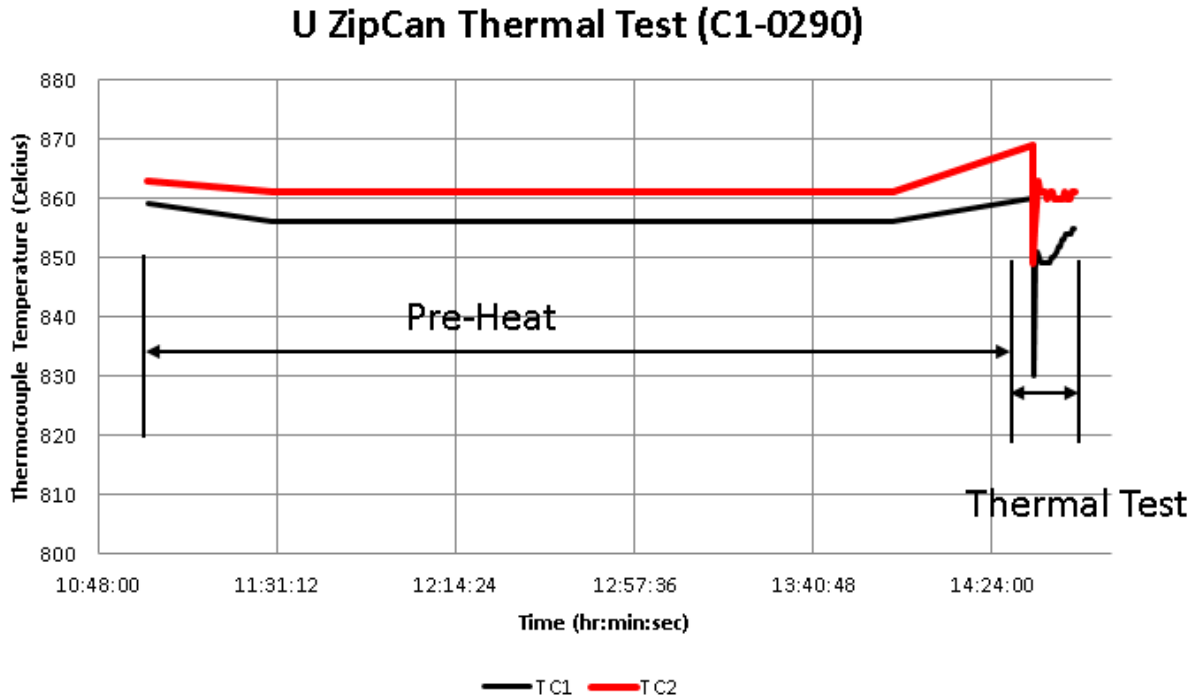


Figure 3.5. Heat test temperature profile.



Figure 3.6. U ZipCan post heat test results.

3.3 LEAK RATE TESTING

3.3.1 Evacuated Envelope (with Back Pressurization)

Leak rate tests that met the test requirements of (49 CFR 173.469 (a)(4)(i)) were performed individually on each test unit before after each special form test. The leak rate tests were performed using ANSI N14.5-2014 *American National Standard for Radioactive Materials – Leakage Tests on Packages for Shipment*, Table A.1, Test Description A.5.5, *Evacuated Envelope (with back pressurization)* and Test Description A.5.6 *Gas bubble techniques*. The American National Standards Institute (ANSI) document indicates that the back-pressure method “. . . is ideal for welded capsules from very small sizes up to the sizes limited by the dimensions of the pressurizing chamber,” and that the “nominal test sensitivity = 10^{-3} - 10^{-8} ref-cm/s” and the bubble test method are used for hermetically sealed test specimens.

Section A.5.5 of ANSI N14.5-1997, *Evacuated Envelope with Helium Back Pressure* of the ANSI document references ASTM E 493, *Standard Test Methods for Leaks Using the Mass Spectrometer Leak Detector in the Inside-Out Testing Mode*. This American Society for Testing and Materials (ASTM) standard provides the method for converting a measured leak rate using the evacuated envelope with the helium back-pressure method into the standardized leak rate that must be compared to the pass/fail criteria specified in 49 CFR 173.469(a)(4)(i), which is 10^{-4} torr-l/s (1.3×10^{-4} atm-cm³/s).

The equation provided in Section 11.1.9 of ASTM E493 is:

$$S_l = (P_e/P_a) \times (1 - e^{(-3600*a*T)}) * (e^{(-a*t)}) \times L \quad (1)$$

where:

- S_l = indicated (measured) leak rate (cc/s)
- P_e = bombing pressure of helium (absolute)
- P_a = atmospheric pressure (absolute)
- T = bombing time (hours)
- t = waiting time between bombing and testing (s)
- L = actual (standardized) leak rate (atm-cc/s)
- a = L/V where V = internal volume
- e = 2.71 (natural logarithm).

Since S_l is being measured and the objective is to solve for L , an iterative solver is required to find the solution. The equation was solved using MS Excel. Note that the ASTM standard uses the term *bombing*, while the ANSI standard uses the term *back-pressure*. These terms are synonymous and are used interchangeably in this report.

To solve Equation 1, the internal volume (i.e., void space) within the test units must be known. For the test units, this internal volume consisted of accessible internal void spaces. Based on the dimensions provided by the drawings, along with queried information from the computer aided design software used to create the drawings (to determine the volume of the irregularly shaped Part #2 shown in Figure 1.1), the void volume is 0.6 cc.

Test units TU-1 (C1-0290) and TU-4 (OPSF1) were leak tested at ORNL by certified ASNT Level II and Level III NDT leak testing personnel using the NDE-70 R.6 procedure. See Appendixes D and E for leak tester certification and the leak testing procedure. The test units were leak tested before and after each special form test. The test apparatuses used for these tests employed a spectrometer tuned to detect helium, a calibrated helium leak to calibrate the system, and two separate vessels—one for helium back

pressurization, and a second one for the subsequent helium leakage rate testing under vacuum conditions. Figure 3.7 provides a schematic of the system used for helium back pressurization, and Figure 3.8 shows a schematic of the system used for the helium leakage rate test. Leak rate test variables and results for TU-1 and TU-4 are shown in Table 3.1.

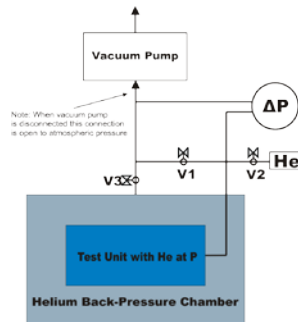


Figure 3.7. Diagram of helium back pressurization test.

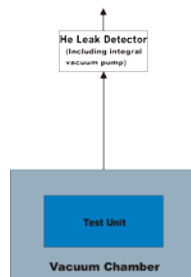


Figure 3.8. Diagram of helium leak testing system.

3.3.2 Gas bubble techniques

The gas bubble test was performed using the methods described in ANSI N14.5-2014, *American National Standard for Radioactive Materials – Leakage Tests on Packages for Shipment*, Table A.1, Test Description A.5.6 (b), Vacuum Bubble. The method involves immersing the test unit in a liquid and then producing a vacuum above the liquid (e.g., water/glycol or isopropyl alcohol) in which the test item is submerged. A leak is indicated by a stream of bubbles (). This method applies to welded capsules. The nominal test sensitivity is 10^{-3} ref-cm³/s (10^{-4} Pa-m³/s). Test units TU-1 (C1-0290) and TU-4 (OPSF1) were bubble tested. Table 3.2 shows the results for each of the tests. See Appendixes D and E for leak tester certification and the leak testing procedure.

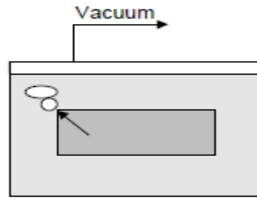


Figure 3.9. Vacuum bubble test.

Table 3.1. Leak rate test variables and results for TU-1 and TU-4

Parameter	Test unit			
	TU-4 (OPSF1)		TU-1 (C1-0290)	
	Leak test 1	Leak test 2	Leak test 1	Leak test 2
Void space – V (cc)	1.057	1.057	1.057	2.2
Bombing pressure – P _e (psig)	30.0	50.0	30	30.0
Atmospheric pressure – P _a (psia)	14.69	14.69	14.69	14.69
Bombing time – T (hr)	0.5	1	0.5	0.5
Time between bombing and testing – t (s)	<3,600	<1,800	3,600	3,600
Measured leak rate (cc/s) – S _l (atm-cc He/s)	2.0×10^{-7}	5.0×10^{-9}	1.7×10^{-7}	7.6×10^{-7}
a = L/V (s ⁻¹)	$<9.46 \times 10^{-5}$	$<9.46 \times 10^{-8}$	$<9.46 \times 10^{-5}$	$<9.46 \times 10^{-5}$
Standardized leak rate – L (atm-cc He/s)	$<1.0 \times 10^{-4}$	$<1.0 \times 10^{-7}$	$<1.0 \times 10^{-4}$	$<1.0 \times 10^{-4}$

Table 3.2. Bubble test results for TU-1 and TU-4

Parameter	Test unit			
	TU-1 (C1-0290)		TU-4 (OPSF1)	
	Bubble test 1	Bubble test 2	Bubble test 1	Bubble Test 2
Bubble test pass/ fail	pass	pass	pass	pass

4. CONCLUSION

Two prototype U ZiPCans were subjected to the tests specified in 49 CFR 173.469 and 10 CFR 71.75 (d)(1)(i), ISO 2919:1999(E), Class 4 impact test. One unit was subjected to the impact test and to pre- and post-leak rate tests, the other unit was subjected to the heat test followed by a leak rate test. Each unit easily surpassed the leak rate criteria following each test, each test specimen did not break or shatter when subjected to the impact test, and the specimen did not melt or disperse when subjected to the heat test. This testing process has shown that the design of the U Zirconia Pre-Encapsulated Canister meets Special Form Criteria.

APPENDIX A. U ZIPCAN DRAWINGS

APPENDIX B. TEST FORMS

Report Number: 2/19/16-2

LEAK TEST REPORT

Test Requested by: J. GARRISSON	Allowable Leak Rate: $< 1.0 E^{-4}$ Std-Atm-cc/s
Work Order Number: 3760A465	Test Pressure Req. Across Boundary: 1 ATM
Item Tested: 2 EA TRIANGLES OPSF-1, OPSF-2	Customer: REDC
Technique Used: BOMB / BELLAR <input checked="" type="checkbox"/> Inside - Out <input type="checkbox"/> Outside - In	Procedure/Rev: NDE 70 R6

EQUIPMENT

LEAK DETECTOR		STANDARD LEAK	
Make and Model: ADIXEN ASM 182 TD+	Manufacturer: VTI	Tracer Gas: He	
Serial Number: HLD 0860905	Model: VSLT-5-3c-He GPPF-7-He-118T	Serial Number: TP860 TP9384	
	Leak Rate: $4.2 E^{-6}$ Atm-cc/s @ -1 atm @ 21.1 °C		
TEST GAUGES		Correlation Formula: $[1 - (T_{cal} - T_{sur}) C_T] LR$	Temp Coefficient: 3.0 %/°C
Temp Gauges: A001067	Due: 6/25/16	Correlated LR: $4.6 E^{-6}$ Atm-cc/s @ -1 atm @ 24.0 °C	
Pressure Gauges: MTE 549	Due:	Calibration Due Date: 9/14/16	

RESULTS Quantitative Semi - Quantitative

MACHINE CALIBRATION		SYSTEM TEST CONDITIONS	
System Pressure: $1.5 E^{-3}$		System Temperature: 24.0 °C	<input checked="" type="checkbox"/> Surface <input type="checkbox"/> Internal Gas
Background: $< 4.0 E^{-10}$ Atm-cc/s		delta P Test Boundary: 1 ATM	
Leak Response: $4.6 E^{-6}$ Atm-cc/s		Tracer Gas: He	% Concentration: CALC
Minimum Detectable Leak: $1.0 E^{-7}$ Atm-cc/s		System Response Time: < 5s	
System Sensitivity: $2.0 E^{-7}$ Atm-cc/s		System Response: $2.0 E^{-9}$ Atm-cc/s	
Response Time: < 5s		Duration of Test: ~ 30s	
Aux. Equipment:			

ACCEPT REJECT SKETCH / DATA ATTACHED w/ stated tracer gas System Leak Rate: $< 1.0 E^{-4}$ Atm cc/s @ -1 atm @ 24.0 °C

COMMENTS:

Pre impact - Oscar m.

Test Conducted By: (Print & Sign Name/Level): E. VIOAC Eric S. Vioac III	Date: 2/19/16	Time: 1:10
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BOMBING TEST REPORT (Supplement)

Leak Test Report Number: 2/19/16-2

Allowable Leak Rate: $< 1.0 \times 10^{-4}$

Item(s) Tested: OPSF-1, OPSF-2

TRACER GAS BOMBING AND LEAK TEST

Bombing Pressure (psig): 30

Tracer Gas: He

Bombing Time: > 30 min

Waiting Time (Sec): < 3600

Internal Volume (cc): 1.057

Measured Leak Rate: 2.0×10^{-9}

Atm cc/s

Calculated Leak Rate: $< 1.0 \times 10^{-4}$ Atm cc/s into vac. @ 24.0°C

Test Results: ACCEPT REJECT CALCULATIONS / DATA ATTACHED

COMMENTS:

Test Conducted By: (Print & Sign Name/Level):

E-VIOAL *Evin S. Vall* LTF

Date:

2/19/16

ASTM Formula	
R	1.00E-05 scc/s
Pe	30 psig
Po	14.696 psia
Ma	28.7 g/mol
M	4 g/mol
T1	1800 sec
T2	3600 sec
V	1.057 cc
L	7.916E-05 scc/s
Part 1	4.329E-04
Part 2	2.427E-02
Part 3	9.520E-01
Rcalc	1.000E-05 scc/s
Lcf	4.0384E-05 scc/s

ASTM/CFR Formula	
L	6.108E-05 scc/s
Lcf	3.1157E-05 scc/s
Rcalc	1.000E-05 scc/s

L (ASNT/MIL)	7.92E-05 scc/s
L (ASTM/CFR)	6.11E-05 scc/s
% Difference	22.8%

$$R = \left[\frac{LP_e}{P_o} \left(\frac{M_A}{M} \right)^{\frac{1}{2}} \right] \times \left\{ 1 - e^{-\left[\frac{LT_1}{VP_o} \left(\frac{M_A}{M} \right)^{\frac{1}{2}} \right]} \right\} \times e^{-\left[\frac{LT_2}{VP_o} \left(\frac{M_A}{M} \right)^{\frac{1}{2}} \right]}$$

$$R = \frac{LP_e}{P_o} \left[1 - e^{-\left(\frac{L}{V} \right) T_1} \right] \times \left[e^{-\left(\frac{L}{V} \right) T_2} \right]$$

- Instructions for Use:**
- 1 - Input estimate for "L" in orange block. This estimate needs only be within a few decades of the expected leak rate. If this calculator fails, the revise estimate closer to measured Q
 - 2 - Input only information in bright yellow blocks specific to test
 - 3 - Tab or select away from the last entered value and press the calculate button for results

OPSF-1, OPSF-2 2/19/16

E. VIOH Senior Analyst LII

Note: formula is modified for uniformity; a = L/V and 3600 converts hours to seconds

LEAK TEST REPORT - BUBBLE TEST

Test Requested by: J. GARRISON	Customer: REDC
Work Order Number: 3760A465	Procedure: NDE 70 R6
Item Tested: 2 EA. TRIANGLES OPSF-1, OPSF-2	Test Pressure Required: 15 INHg
Technique Used: VAC BOX	Liquid Media Used: IMMERSIT CIM 150
Test Gas Used: VAC	Liquid Applicator Type: IMMERSION
Inspection Light Intensity: >100 FC	Post Cleaning Method: DEMIN H2O RINSE
Other Apparatus Used: FLASHLIGHT	

Direct Pressure Technique Vacuum Pressure Technique

Component Limits of Test:

Component Test Site BLDG 5500 Component Installation Site

Gauges				Test Pressure		Temperature	
Mfg	ID No	Calibration Date	Range	Beginning	End	Beginning	End
	A002126	8/26/15	0-30 INHg	15 INHg	15 INHg	AMBIENT	AMBIENT

Temperature Measuring Device

Mfg. —	Model —	Range —	I.D. Number —
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RESULTS ACCEPT REJECT POST CLEANING PERFORMED: Y N

Comments: AMBIENT SHOP AIR ~ 70°F

Test Conducted By: (Print & Sign Name/Level): *EVIDAL Eric S Wall LTD* Date: 2/22/16

TEST FORM 1 – Impact Test ISO 2919

Test Plan ORNL/NTRC-067

Test Unit OPSF-1

VERIFIED

TASK

✓

The weight of the impact billet has been measured and verified to be 2 kg or greater:

Measured weight of billet 2.000 (kg)

Scale used for measurement: Mettler Toledo Calibration due: Metrology Scale

✓

The calibration of the 1-m ruler has been verified:

1-m Ruler Equipment # A 001146 Calibration due: 11/3/16

✓

The test unit with supporting device has been placed (centered) on NTRC indoor drop pad.

✓

The drop test release mechanism has been attached to the crane.

✓

The impact billet has been captured by the release mechanism.

✓

The billet has been centered over the test unit and a picture has been taken.

✓

The billet has been raised to height of 1 meter over the highest point of the test unit and a picture has been taken.

✓

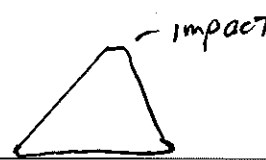
The billet was released and impacted the test unit.

✓

All observable damage to the test unit caused by the impact test has been recorded and pictures of the test unit after the impact test have been taken.

Comments:

Weight of Test Unit = 57g
Vertex Impact



I certify that the above tasks have been performed and that the observations and comments are correct.

[Signature]
Testing Technician

3/2/2016
Date

Andrea Beuth
Checked by

4/1/16
Date

*All photographs will be uniquely identified with test unit, date and time to ensure that the proper sequence can be reconstructed

Report Number: 3/9/16-2

LEAK TEST REPORT

Test Requested by: C. BLESSINGETZ	Allowable Leak Rate: $\leq 1.0E^{-7}$	Std-Atm-cc/s
Work Order Number: 3760A465	Test Pressure Req. Across Boundary: 1 ATM	
Item Tested: OPSF-1	Customer: REDC	
Technique Used: BOMB/BELL JAR	<input checked="" type="checkbox"/> Inside - Out <input type="checkbox"/> Outside - In	Procedure/Rev: NDE 70 R.C

EQUIPMENT

LEAK DETECTOR		STANDARD LEAK	
Make and Model: ADIXEN ASM 182 TD+	Manufacturer: VTI	Tracer Gas: He	
Serial Number: HLD 0860905	Model: GPPT-HE-118T	Serial Number: TP5754	
		Leak Rate: $5.37E^{-8}$ Atm-cc/s @ -1 atm @ 23.4 °C	
TEST GAUGES		Correlation Formula: $[1 - (T_{cal} - T_{surf}) C_T] LR$	Temp Coefficient: 2.0 %/°C
Temp Gauges: A001952	Due: 6/10/16	Correlated LR: $5.36E^{-8}$ Atm-cc/s @ -1 atm @ 23.3 °C	
Pressure Gauges: MTE 549	Due: —	Calibration Due Date: 02/23/17	

RESULTS

Quantitative Semi - Quantitative

MACHINE CALIBRATION		SYSTEM TEST CONDITIONS	
System Pressure: $1.5E^{-3}$ mb		System Temperature: 23.4 °C	<input checked="" type="checkbox"/> Surface <input type="checkbox"/> Internal Gas
Background: $7.3E^{-11}$ Atm-cc/s		delta P Test Boundary: 1 ATM	
Leak Response: $5.4E^{-8}$ Atm-cc/s		Tracer Gas: He	% Concentration: RALC
Minimum Detectable Leak: $1.0E^{-9}$ Atm-cc/s		System Response Time: < 5s	
System Sensitivity: $2.0E^{-9}$ Atm-cc/s		System Response: $5.0E^{-9}$ Atm-cc/s	
Response Time: < 5s		Duration of Test: ~ 1 min	

Aux. Equipment:

ACCEPT REJECT SKETCH / DATA ATTACHED

System Leak Rate: $< 1.0E^{-7}$ Atm cc/s @ -1 atm @ 23.4 °C w/ stated tracer gas

COMMENTS:

FINE LT - POST DROP TEST / IMPACT TEST

Test Conducted By: (Print & Sign Name/Level): E. VIOAL Eric S Vioal LTI	Date: 3/9/16	Time: 3:00
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BOMBING TEST REPORT (Supplement)

Leak Test Report Number:

3/9/16-2

Allowable Leak Rate:

$\leq 1.0 \times 10^{-7}$

Item(s) Tested:

OPSF-1

TRACER GAS BOMBING AND LEAK TEST

Bombing Pressure (psig):

50

Tracer Gas:

He

Bombing Time:

> 1 Hr

Waiting Time (Sec):

< 1800

Internal Volume (cc):

1.057

Measured Leak Rate:

5.0×10^{-9}

Atm cc/s

Calculated Leak Rate:

$< 1.0 \times 10^{-7}$ Atm cc/s into vac. @ 23.4 °C

Test Results:

ACCEPT

REJECT

CALCULATIONS / DATA ATTACHED

COMMENTS:

Test Conducted By: (Print & Sign Name/Level):

E.VIOAC Eric S. Vail LTL

Date:

3/9/16

ASNT Formula	
R	5.00E-07 sec/s
Pe	50 psig
Po	14.696 psia
Ma	28.7 g/mol
M	4 g/mol
T1	3600 sec
T2	1800 sec
V	1.057 cc
L	9.450E-06 sec/s
Rcalc	5.022E-07 sec/s
Lcf	6.672E-06 sec/s

ASTM/CFR Formula	
L	6.664E-06 sec/s
Lcf	4.7051E-06 sec/s
Rcalc	5.030E-07

RESULTS
 L (ASNT/MIL) 9.45E-06 sec/s
 L (ASTM/CFR) 6.66E-06 sec/s
 % Difference 29.5%

$$R = \left[\frac{LP_e}{P_o} \left(\frac{M_A}{M} \right)^{\frac{1}{2}} \right] \times \left\{ 1 - e^{-\left[\frac{LT_1}{VP_o} \left(\frac{M_A}{M} \right)^{\frac{1}{2}} \right]} \right\} \times e^{-\left[\frac{LT_2}{VP_o} \left(\frac{M_A}{M} \right)^{\frac{1}{2}} \right]}$$

$$R = \frac{LP_e}{P_o} \left[1 - e^{-\left(\frac{L}{V} \right) T_1} \right] \times \left[e^{-\left(\frac{L}{V} \right) T_2} \right]$$

Note: formula is modified for uniformity; a = L/V and 3600 converts hours to seconds

- Instructions for Use:**
- 1 - Input estimate for "L" in orange block. This estimate needs only be within a few decades of the expected leak rate. If this calculator fails, the revise estimate closer to measured Q
 - 2 - Input only information in bright yellow blocks specific to test
 - 3 - tab or select away from the last entered value and press the calculate button for results

OPSF-1
 3/9/16

LEAK TEST REPORT

Test Requested by: D. GARRISON	Allowable Leak Rate: $< 1.0 \times 10^{-4}$ Std-Atm-cc/s
Work Order Number:	Test Pressure Req. Across Boundary: 1 ATM
Item Tested: 4 EA. RSTD SPECIAL FORM CAPSULES	Customer: RSTD
Technique Used: BOMB / BELL JAR	<input checked="" type="checkbox"/> Inside - Out <input type="checkbox"/> Outside - In Procedure/Rev: NDE 70 R.6

EQUIPMENT

LEAK DETECTOR		STANDARD LEAK	
Make and Model: ADIXEN ASM 340	Manufacturer: VTI	Tracer Gas: He	
Serial Number: HL0 1601393	Model: VSLT-5-3C-He	Serial Number: TP860	
		Leak Rate: 4.82×10^{-6} Atm-cc/s @ -1 atm @ 22.1 °C	
TEST GAUGES		Correlation Formula: [1 - (T _{cal} - T _{sur}) C _T] LR	Temp Coefficient: 3.0 %/°C
Temp Gauges: A001957	Due: 6/22/17	Correlated LR: 4.56×10^{-6} Atm-cc/s @ -1 atm @ 20.3 °C	
Pressure Gauges: MTE549	Due: -	Calibration Due Date: 9/9/17	

RESULTS Quantitative Semi - Quantitative

MACHINE CALIBRATION		SYSTEM TEST CONDITIONS	
System Pressure: 5.0 × 10⁻³ mb		System Temperature: 20.3 °C	<input checked="" type="checkbox"/> Surface <input type="checkbox"/> Internal Gas
Background: 2.0 × 10⁻¹⁰ Atm-cc/s		delta P Test Boundary: -1 ATM	
Leak Response: 4.5 × 10⁻⁶ Atm-cc/s		Tracer Gas: He	% Concentration: CALC
Minimum Detectable Leak: 1.0 × 10⁻⁷ Atm-cc/s		System Response Time: ~ 1 MIN	
System Sensitivity: 2.0 × 10⁻⁷ Atm-cc/s		System Response: 1.7 × 10⁻⁷ Atm-cc/s	
Response Time: ~ 10s		Duration of Test: 90s	

Aux. Equipment:

ACCEPT REJECT SKETCH / DATA ATTACHED System Leak Rate: $< 1.0 \times 10^{-4}$ Atm cc/s @ **-1 atm @ 20.3 °C**
w/ stated tracer gas

COMMENTS:

S/N: **C1-0286, 288, 289, 290** **FINE LT**

Test Conducted By: (Print & Sign Name/Level): E. VIOAR [Signature] LII	Date: 11/22/16	Time: 1:15
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BOMBING TEST REPORT (Supplement)

Leak Test Report Number: 11/22/16-1

Allowable Leak Rate: $< 1.0 \text{ E}^{-4}$

Item(s) Tested: 4 EA. SPECIAL FORM CAPSULES

TRACER GAS BOMBING AND LEAK TEST

Bombing Pressure (psig): 30

Tracer Gas: He

Bombing Time: $> 1800 \text{ s}$

Waiting Time (Sec): $< 3600 \text{ s}$

Internal Volume (cc): 1.057

Measured Leak Rate: 1.7 E^{-7}

Atm cc/s

Calculated Leak Rate: $< 1.0 \text{ E}^{-4}$ Atm cc/s into vac. @ $20.3 \text{ }^\circ\text{C}$

Test Results: ACCEPT REJECT CALCULATIONS / DATA ATTACHED

COMMENTS:

Test Conducted By: (Print & Sign Name/Level):

E. VIOAL Eric S. Vioal LTI

Date:

11/22/16

LEAK TEST REPORT - BUBBLE TEST

Test Requested by: J. GARRISON	Customer: RSTD
Work Order Number:	Procedure: NDE 70 R.6
Item Tested: 4 EA. SPECIAL FORM CAPSULES	Test Pressure Required: -15" Hg
Technique Used: VAC BOX	Liquid Media Used: IMMERSIT CIM 200 @ 20% SOLW
Test Gas Used: VAC	Liquid Applicator Type: IMMERSION
Inspection Light Intensity: > 100 FC	Post Cleaning Method: RINSE/WIPE
Other Apparatus Used: FLASHLIGHT	

Direct Pressure Technique Vacuum Pressure Technique

Component Limits of Test:

Component Test Site **7606 A** Component Installation Site **-**

Gauges				Test Pressure		Temperature	
Mfg	ID No	Calibration Date	Range	Beginning	End	Beginning	End
GAST	A002124	8/11/16	0-30 IN Hg	-15 IN Hg	-15 IN Hg	20.3°C	20.3°C

Temperature Measuring Device

Mfg. OMEGA	Model HH804	Range K-TYPE	I.D. Number A001951
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RESULTS ACCEPT REJECT POST CLEANING PERFORMED: Y N

Comments: **C1-0286, 288, 289, 290**

Test Conducted By: (Print & Sign Name/Level): E. VIOLE Eric S. Viole LII	Date: 11/22/16
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TEST FORM 2 – Thermal Test Checklist

Test Plan ORNL/NTRC-074
Rev. 0

Test Unit 1-C10290

VERIFIED

TASK

- The test unit tray has been placed in the furnace.
- Two calibrated Type K thermocouples have been installed in the working area of the furnace and attached to the Fluke thermocouple reader.
Fluke Equipment # B1332 Calibration Due: 9/27/2017
- The furnace doors has been closed and the furnace has been turned on with a set point of 850° C.
Furnace Equipment # 18334 Calibration Due: No cal record
- Thermocouple readings have been made every 30 minutes for at least 3 hours.
- Any changes in the furnace set point during the three-hour preheat period have been recorded on TEST FORM 5.
- Just prior to test unit insertion, a final preheat temperature recording was made.
- The furnace door has been opened, the test unit inserted, the furnace door closed and the furnace activated with a set point of 850° C (1560° F) (or as adjusted during the preheat process).
- When both thermocouple readings have reached 800° C (1475° F), the 10-minute thermal test was started.
- Thermocouple readings were taken every 30 seconds for the duration of the 10-minute thermal test.
- Adjustments were made to the furnace set point as directed by the test director.
- When the 10-minute test period was finished, the furnace was turned off and furnace door was opened to the maximum extent possible.
- As soon as conditions permitted, the test unit was removed from the furnace and allowed to cool naturally.
- Any deformation or other unusual circumstances regarding the test or the test unit was recorded.

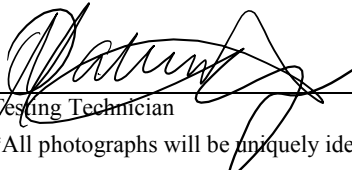
Comments:

C1-0290 Furnace Hood IE 960

Set - 850°C TC's and Fluke are

TC1-856°C TC2-861°C calibrated ✓

I certify that the above tasks have been performed and that the observations and comments are correct.

 1/25/17 Matthew R Feldman 8/18/2017
Testing Technician Date Checked by Date

*All photographs will be uniquely identified with test unit, date and time to ensure that the proper sequence can be reconstructed

TEST FORM 3 – Thermal Test Preheat Data Sheet

Test Plan ORNL/NTRC-074
 Rev. 0

Test Unit 1- C1-0290

VERIFIED

TASK



Record the temperature in the furnace every thirty (30) minutes for the duration of the preheat (at least 3 hours):

Time	Thermocouple 1 (°C)	Thermocouple 2 (°C)
11:00	859	863
11:30	856	861
12:00	856	861
12:30	856	861
1:00	856	861
1:30	856	861
2:00	856	861

Comments: Three hour pre heat above 850°C
Thermocouple 1 = BF3874
Thermocouple 2 = BF3F05

I certify that the above tasks have been performed and that the observations and comments are correct.

 1/27/17 Matthew R Feldman 8/18/2017
 Testing Technician Date Checked by Date

*All photographs/movies will be uniquely identified with test unit, date and time to ensure that the proper sequence can be reconstructed

TEST FORM 4 – Thermal Test Data Sheet

Test Plan ORNL/NTRC-074
Rev. 0

Test Unit C1-0290

VERIFIED
✓

TASK

Record the temperature in the furnace every 30 seconds for the duration of the test:

Time	Thermocouple 1 °C	Thermocouple 2 °C
0	830	849
30	851	855
100	851	863
130	850	861
200	849	861
230	849	861
300	849	861
330	849	861
400	849	860
430	850	861
500	850	860
530	851	860
600	852	860
630	852	860
700	853	860
730	853	861
800	854	860
830	854	860
900	854	860
930	855	861
1000	855	861

Furnace @
850°C

Comments: 1/26/2017 @ 2:34 pm (start)
Tu pillowed after test. TCI = BF3874 TCC = BF3F05

I certify that the above tasks have been performed and that the observations and comments are correct.

Testing Technician [Signature] Date 1/27/17 Checked by Matthew R. Feldman Date 8/18/2017

*All photographs/movies will be uniquely identified with test unit, date and time to ensure that the proper sequence can be reconstructed

LEAK TEST REPORT

Test Requested by: <u>J. GARRISON</u>	Allowable Leak Rate: <u>$< 1.0 \times 10^{-4}$</u> Std-Atm-cc/s
Work Order Number:	Test Pressure Req. Across Boundary: <u>1 ATM</u>
Item Tested: <u>1 EA. TRIANGLE C1-0290</u>	Customer: <u>NSW</u>
Technique Used: <u>BOMB/BELL JAR</u>	<input checked="" type="checkbox"/> Inside - Out <input type="checkbox"/> Outside - In Procedure/Rev: <u>NDE 70 R.6</u>

EQUIPMENT

LEAK DETECTOR		STANDARD LEAK	
Make and Model: <u>ADIXEN ASM 340</u>	Manufacturer: <u>VTI</u>	Tracer Gas: <u>He</u>	
Serial Number: <u>HLD 1601393</u>	Model: <u>VSLT-5-3C-He</u>	Serial Number: <u>TP860</u>	
		Leak Rate: <u>4.82×10^{-6}</u> Atm-cc/s @ <u>-1</u> atm @ <u>22.1</u> °C	
TEST GAUGES		Correlation Formula: $[1 - (T_{cal} - T_{surf}) C_T] LR$	
Temp Gauges: <u>A001952</u>	Due: <u>6/22/17</u>	Temp Coefficient: <u>3.0</u> %/°C	
Pressure Gauges: <u>MTE 549</u>	Due: <u>-</u>	Correlated LR: <u>4.08×10^{-6}</u> Atm-cc/s @ <u>-1</u> atm @ <u>17.0</u> °C	
		Calibration Due Date: <u>9/9/16</u>	

RESULTS

Quantitative Semi - Quantitative

MACHINE CALIBRATION		SYSTEM TEST CONDITIONS	
System Pressure: <u>1.0×10^{-2} mb</u>		System Temperature: <u>17</u> °C	<input checked="" type="checkbox"/> Surface <input type="checkbox"/> Internal Gas
Background: <u>1.2×10^{-9}</u> Atm-cc/s		delta P Test Boundary: <u>-1 ATM</u>	
Leak Response: <u>4.2×10^{-6}</u> Atm-cc/s		Tracer Gas: <u>He</u>	% Concentration: <u>CALL</u>
Minimum Detectable Leak: <u>1.0×10^{-7}</u> Atm-cc/s		System Response Time: <u>90 s</u>	
System Sensitivity: <u>2.0×10^{-7}</u> Atm-cc/s		System Response: <u>7.6×10^{-7}</u> Atm-cc/s	
Response Time: <u>~5 s</u>		Duration of Test: <u>90 s</u>	
Aux. Equipment:			

ACCEPT REJECT SKETCH / DATA ATTACHED

System Leak Rate: $< 1.0 \times 10^{-4}$ Atm cc/s @ -1 atm @ 17 °C
w/ stated tracer gas

COMMENTS:

FINE LT
POST HEAT TEST

Test Conducted By: (Print & Sign Name/Level): <u>E. VIDAL Eric S Vidal LTI</u>	Date: <u>2/1/17</u>	Time: <u>1:40</u>
---	------------------------	----------------------

BOMBING TEST REPORT (Supplement)

Leak Test Report Number: 2/1/17-1

Allowable Leak Rate: $< 1.0 \times 10^{-4}$

Item(s) Tested:

1 EA. TRIANGLE C1-0290

TRACER GAS BOMBING AND LEAK TEST

Bombing Pressure (psig): 30

Tracer Gas: He

Bombing Time: 30 MIN

Waiting Time (Sec): < 3600 s

Internal Volume (cc): * 1.057 (PRE-HT)

Measured Leak Rate: 7.6×10^{-7}

Atm cc/s

Calculated Leak Rate: $< 1.0 \times 10^{-4}$ Atm cc/s into vac. @ 17 °C

Test Results:

ACCEPT

REJECT

CALCULATIONS / DATA ATTACHED

COMMENTS:

* ESTIMATED 2.2 cc USED FOR CALC

Test Conducted By: (Print & Sign Name/Level):

E-VIOAC Eric S. Veibel LII

Date:

2/1/17

LEAK TEST REPORT - BUBBLE TEST

Test Requested by: J. GARRISON	Customer: NGIT
Work Order Number:	Procedure: NDE 70 R6
Item Tested: 1 EA TRIANGLE C1-0290	Test Pressure Required: 15" Hg
Technique Used: VAC BOX	Liquid Media Used: IMMERSIT CIM 200 @ 20%
Test Gas Used: VAC	Liquid Applicator Type: IMMERSION
Inspection Light Intensity: >100 FC	Post Cleaning Method: DIPWASH
Other Apparatus Used: FLASHLIGHT	

Direct Pressure Technique Vacuum Pressure Technique

Component Limits of Test:

Component Test Site 7606 A Component Installation Site -

Gauges			Test Pressure		Temperature		
Mfg	ID No	Calibration Date	Range	Beginning	End	Beginning	End
	A002124	8/11/16	0-30" Hg	15" Hg	15" Hg	17°C	17°C

Temperature Measuring Device

Mfg. OMEGA	Model HH804	Range K-TYPE	I.D. Number A001952
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RESULTS ACCEPT REJECT POST CLEANING PERFORMED: Y N

Comments: POST HEAT TEST

Test Conducted By: (Print & Sign Name/Level): E. VIOAL Luis S Vidal LTF Date: 2/1/17

APPENDIX C. WELD INSPECTION REPORT

WELD INSPECTION REPORT

DATE
12/6/2016

DRAWING TITLE
Triangle Encasement Components (SI Units Version)

REPORT NUMBER

SYSTEM
C1-0288, C1-0289, C1-0290

DRAWING NUMBER
SK-NMP-20160616-01

WORK ORDER NUMBER

WELD NUMBER
W-1

SHOP
REDC 7930

INSP. SPEC.
FHRD-T-NDE 21 Rev 2

WELD SPEC.
GT88-A (PP), *GT88-1 (PP)

WELD PROCESS
GTAW

JOINT TYPE
 FILLET SINGLE WELDED DOUBLE WELDED

BASE MATERIAL(S)				FILLER MATERIAL(S)			
TYPE	304/304L SS	304/304L SS		TYPE	308L		
IR NO.	N/A	N/A		IR NO.	N/A		
HEAT NO.	177228	843690		HEAT NO.	DACU		
MFG.	Ta Chen International	ATI		MFG.	Techalloy		
FORM	.500" Plate ASTM A240-2015	.0600" Plate ASTM A240-13C		SIZE	.045"		
PART	1	2					
PIECE	Triangle Encasement Base	Triangle Encasement Top					
SIZE	See Drawing	See Drawing					

JOINT PREPARATION
Accept

FIXTURE
Copper Fixture

WELDER
B Leffew

CRAFTSMAN
Jay Kehn

CLEANER

ACETONE ALCOHOL

WELDER

CRAFTSMAN

INERT GAS

COVER

ARGON CFH 25

HELIUM CFH _____

MIXED CFH _____

BACKUP

ARGON CFH glovebox

HELIUM CFH _____

FIT UP
Accept

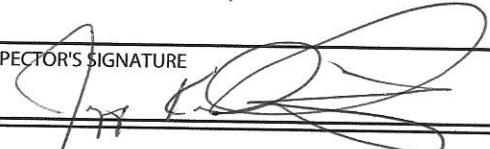
WORK CONDITION
Accept

STAGE	ROOT PASS	SECOND LAYER	INTERM.	FINAL	PREHEAT TREATMENT	POSTHEAT TREATMENT
INSP.						
VISUAL	/	/	/	SAT	N/A	N/A
PENETRANT	/	/	/	/	BATCH NO.	SKL-SP1 SKC-S SKD-S2
RADIOGRAPH	/	/	/	/	ULTRASONIC	/

MACHINE WELDING VARIABLES VOLTS **7-17** AMPS **5-50** INTERPASS TEMP. **N/A**

REPAIRS

REMARKS
***Weld filler metal used only if necessary.**

INSPECTOR'S SIGNATURE  DATE **12/6/2016** FINAL ACCEPTANCE DATE

APPENDIX D. LEAK TESTER CERTIFICATION



The American Society for Nondestructive Testing, Inc.

Be it known that

Jeff M Pryor

has met the established and published Requirements for Certification by ASNT as

NDT Level III


in the following Nondestructive Testing Methods:

<u>Method</u>	<u>Issue Date</u>	<u>Expiration Date</u>
Leak Testing	6/15	6/20
Liquid Penetrant Testing	6/15	6/20
Magnetic Particle Testing	6/15	6/20
Radiographic Testing	6/15	6/20
Ultrasonic Testing	6/15	6/20
Visual Testing	6/15	6/20



126138

Certificate Number


ASNT President


Certification Management Council Chair

This certificate is the property of ASNT, is not official without ASNT's raised gold seal and is subject to revocation prior to the listed expiration date.
This certificate should be verified on the ASNT website or by contacting the ASNT Technical Services Department.

APPENDIX E. LEAK TESTING PROCEDURE

ORNL Leak test procedure not available for public release. Procedure number is NDE 70, Rev. 6

APPENDIX F. CALIBRATION RECORDS

OAK RIDGE NATIONAL LABORATORY

METROLOGY DEPARTMENT

TEST REPORT

ITEM: 39.38" LENGTH STANDARD

Serial Number: A001146

CUSTODIAN: M. FELDMAN

	AS FOUND	REQUIREMENT	INSPECTION METHOD
LENGTH	39.4000/39.4099	39.38	CMM

Temperature: 68 ° F

Date 11/03/11

Inspector '024294

Reviewed by

Ben X. Symon 28642

11/3/11

Date Due 11/03/16

NATIONAL INSTITUTE OF STANDARDS AND TECHNOLOGY TRACEABILITY
ESTABLISHED THROUGH ORNL PRIMARY STANDARDS

Standards used:

ID#

Calibration Due Date

M212632

5/19/14



Certificate of Calibration

ISO 9001:2008 (10101/2)

Everett Service Center

Certificate Number: 291652		
Data Type: Found-Left		Calibration Date: 07-Nov-2016
Result Summary: In Tolerance		Calibration Due: 07-Nov-2017
Manufacturer: Fluke		Certificate Date: 07-Nov-2016
Model: 52 II		Temperature: 24.2 °C
Serial Number: 36370410WS		Humidity: 35.9 %
Description: Thermometer		

Procedure: Fluke 52-II:(1 YEAR) ZCAL VER /5520	Revision: 1.2
Customer: MCMaster-CARR SUPPLY COMPANY	
City: DOUGLASVILLE	Country: US
State: GA	
Purchase Order: COC	RMA: 31143929

This calibration is traceable to the International System of Units (SI), through National Metrology Institutes (NIST, PTB, NRC, NPL, etc.), radiometric techniques, or natural physical constants. This certificate applies only to the item identified and shall not be reproduced other than in full, without the specific written approval by Fluke Corporation. Calibration certificates without signature are not valid. The calibration has been completed in accordance with Fluke Electronics Corporation Quality System Document 111.0 Revision 118 8/2014 and/or Fluke 17025 Quality Manual QSD 111.41 Revision 005 9/2014.

The Data Type found in this certificate must be interpreted as:

- As - Found Calibration data collected before the unit is adjusted and / or repaired.
- As - Left Calibration data collected after the unit has been adjusted and / or repaired.
- Found-Left Calibration data collected without any adjustment and / or repair performed.



Cert #: 291652
 Date: 07-Nov-2016
 Due: 07-Nov-2017
 www.fluke.com


 JACOB MILLER
 Issued By

Certificate Number: 291652

Date of Calibration: 07-Nov-2016

Standards Used

Asset	Description	Cal-Date	Cal-Due
B1322	Fluke 5520A Calibrator	27-Sep-2016	27-Sep-2017

Job# 3054371

Date: 1/24/17

**Technical Support Department
Instrument Data Continuation Sheet**

Tech: 30220

Std: A001277

M210101

A002021

Furnace	Standard	UUT Reading			
		BF3874	Error	BF3F05	Error
21.5	21.5	20.9	-0.6	21.1	-0.4
750.0	754.1	755.0	0.9	754	-0.1
800.0	803.5	804.5	1.0	803.6	0.1
850.0	854.0	855.0	1.0	854.4	0.4
900.0	904.3	905.4	1.1	905.1	0.8
950.0	955.4	956.9	1.5	956.3	0.9



Calibration Results

Oak Ridge National Laboratory

ORNL Metrology Laboratory
Bethel Valley Rd. Bldg. 5510A
Oak Ridge, TN 37831-6366

Unit Under Test Information

Manufacturer: Oak Ridge National Laboratory
Description: Type S Thermocouple Dual Junction
Model Number: N/A
Serial Number: N/A
Asset / ID Number: A001277
Custodian: Anthony D Mcbee
Work Order Number: 2016002594

Customer Information

Anthony D Mcbee
Building: 2547
Room: 002
Mail Stop: 6300
865-574-6293

Test Information

Certificate Number: 2016002594
Overall Result: Pass
Performed on: 1/18/2017
Next Cal Due: 1/18/2018
Performed by: Greg Strickland
Environment: 23.4°C 46.8%Rh
Received: In Tolerance

Notes:

Asset No.



Work Order No.



ORNL Metrology Laboratory (ORNL ML) certifies that the above listed instrument meets or exceeds all specifications as stated in the referenced procedure unless otherwise noted. This Report of Calibration applies only to the item being calibrated, identified above.

This calibration report documents the traceability to national standards, which realize the units of measurement according to the International System of Units (SI). Calibration data and conformity assessment (Pass/Fail decision) is limited to the performance of the instrument at the time of test. The "Next Cal Due" date is based on manufacturer's recommendations or best calibration practices and with customer agreement (in the case of external ORNL customers); the instrument should not be used past this date without recalibration. This report shall not be reproduced, except in full, unless written permission for an approved abstract is obtained from ORNL ML. Any report containing accredited data shall not be used to claim product certification, approval, or endorsement by NVLAP, NIST, or any agency of the Federal Government. Calibration reports without authorizing signature(s) are not valid.

For accredited data, measurement uncertainties at the time of test, expressed in base units, are given on the following pages, where applicable. They are calculated in accordance with the methods described in EA-4/02, NIST TN1297, DKD-3, or other applicable documents that comply with the Guide to the Uncertainty in Measurement (GUM), using a coverage factor of $k=2$, corresponding to a confidence level of approximately 95%. Unless otherwise indicated, any conformity determination in this report is based on a Test Uncertainty Ratio (TUR) of 4:1 or greater. Any TUR less than 4:1 will be identified in the test data. It is the responsibility of the instrument custodian, with the assistance of his/her Quality Representative, to determine whether this level of confidence for the determination of conformance is adequate for the intended use of this instrument.

This calibration was performed using measurement standards traceable to the appropriate standard(s), maintained by the National Institute of Standards and Technology (NIST), to accepted intrinsic standards of measurement, or is derived by ratio type self-calibration techniques. The calibration system used to derive accredited data complies with the requirements of NIST Handbook 150, ANSI/NCSL Z540.1-1999 (R2002), ISO/IEC 17025.

Standards Used

ID	Description	Service Date	Due Date
0078611	Isotech MicroK-100 Thermometry Bridge	6/2/2016	6/2/2017
0078621	Isotech ITL-M-17673 Silver Freeze Point Cell	9/4/2009	9/4/2017
A001412	Rosemount 162CE SPRT	11/15/2016	2/15/2017

FOUND_LEFT

Procedure used: Manual Data File Reader, Rev. 1.0

Test Data									
UUT Range / Comment	Standard Reading	Standard Modifier	UUT Reading	UUT Tolerance	UUT Error	% Tol	Measurement Uncertainty	Accred	Test Status

INITIAL INSPECTION

No Calibration Seals found on the UUT.
 Instrument was received in good, functional condition.
 Procedure used: Manual Data

UUT Specification is based on (Type S Special Grade +/- 0.6 Deg C or 0.1 % WIG Plus Indicator Specification of +/- 0.6 Deg C)

Standard Temperature (Deg C)	UUT Temperature (Deg C)	UUT Error (Deg C)	UUT Specification (Deg C)	% TOL	Measurement Uncertainty (Deg C)	Result
A001277-A						
231.97	231.7	-0.3	1.20	27	3.1E-01	*Pass
418.97	419.0	0.0	1.20	1	3.1E-01	*Pass
594.01	593.2	-0.8	1.26	67	1.0E+00	*Pass
961.78	961.2	-0.5	1.56	35	1.0E+00	*Pass
A001277-B						
231.99	231.7	-0.3	1.20	25	3.1E-01	*Pass
419.05	419.1	0.0	1.20	2	3.1E-01	*Pass
594.01	593.1	-0.9	1.26	73	1.0E+00	*Pass
961.78	961.3	-0.5	1.56	33	1.0E+00	*Pass

* Test Uncertainty Ratio < 4:1

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-- End of measurement results--

Approved By: Greg Strickland 1/18/2017  
 Technical Manager