

ADVANTG—An Automated Variance Reduction Parameter Generator



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Reactor and Nuclear Systems Division

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PARAMETER GENERATOR**

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DEFINITIONS, ACRONYMS, AND ABBREVIATIONS

ADVANTG	AutomateD VArIaNce reducTion Generator
CADIS	Consistent Adjoint Driven Importance Sampling, a method for generating variance reduction parameters to accelerate the estimation of an individual tally.
Denovo	3-D, block-parallel discrete ordinates transport code developed at Oak Ridge National Laboratory.
deterministic	Methods (e.g., the discrete ordinates method) and codes that discretize the independent variables of the transport equation and solve the resulting linear algebraic systems of equations via iterative methods.
FW-CADIS	Forward-Weighted CADIS, a method for generating variance reduction parameters to obtain relatively uniform statistical uncertainties across multiple tally regions or energy bins.
FOM	Tally figure of merit, calculated as $1/(R^2T)$, where R is the tally relative error and T is the simulation run time in minutes.
GL	Gauss-Legendre, a type of product quadrature.
GMRES	Generalized Minimum RESidual, a Krylov subspace method for iteratively solving linear algebraic systems of equations.
LD	Linear discontinuous, a discretization scheme that expands the angular flux within a voxel in terms of a volume-average value and a slope in the x , y , and z dimensions.
LDFE	Linear-discontinuous finite element, a type of triangular quadrature.
MCNP	Monte Carlo N-Particle. A continuous-energy Monte Carlo transport code developed at Los Alamos National Laboratory.
Monte Carlo	Methods and codes that simulate particle transport by stochastically sampling individual particle events (e.g., emission from source, free-streaming between collisions, collision kinematics) and tallying the average behavior.
PBS	Portable Batch System, a job scheduling system used in Linux cluster environments
Python	Open-source scripting language (https://www.python.org/).
QR	Quadruple range, a type of product quadrature.
SDEF	MCNP fixed-source specification
Silo	Open-source mesh and field library and scientific database originally developed at Lawrence Livermore National Laboratory (https://wci.llnl.gov/simulation/computer-codes/silo).
S_N	discrete ordinates
SWORD	SoftWare for Optimization of Radiation Detectors, a graphical user interface and framework for constructing and evaluating radiation detection systems developed by the U.S. Naval Research Laboratory. The software is distributed through RSICC as package CCC-767.

TLD	Trilinear discontinuous, a discretization scheme that is similar to LD, but is based on an angular flux expansion that also includes the xy , yz , xz , and xyz cross terms.
Trilinos	Collection of open-source software packages developed at Sandia National Laboratory (https://trilinos.org/). Includes packages for solving linear systems of equations using modern iterative methods.
VisIt	Open-source 3-D, parallel visualization tool originally developed at Lawrence Livermore National Laboratory (https://wci.llnl.gov/simulation/computer-codes/visit/).
WWINP	MCNP weight-window input file
ZAID	MCNP nuclide identification number. ZAIDs are written in the form $ZZZAAA$, where ZZZ is a one- to three-digit atomic number and AAA is a three-digit mass number.

1. INTRODUCTION

The Automated Variance Reduction Generator (ADVANTG) software automates the process of generating variance reduction parameters for continuous-energy Monte Carlo simulations of fixed-source neutron, photon, and coupled neutron-photon transport problems using MCNP5 (X-5 Monte Carlo Team 2003). ADVANTG generates space- and energy-dependent mesh-based weight-window bounds and biased source distributions using three-dimensional (3-D) discrete ordinates (S_N) solutions of the adjoint transport equation that are calculated by the Denovo package (Evans et al. 2010). ADVANTG outputs weight-window lower bounds as an MCNP-compatible weight-window input (WWINP) file. Weight-window control parameters and biased source distributions are output as WWP and SB cards, respectively, in an extended version of the user's original MCNP input file.

ADVANTG also provides the capability to execute discrete ordinates calculations without generating variance reduction parameters. ADVANTG extracts problem geometry, material composition, source, and tally information from either an MCNP5 model or from a model created using the SWORD software (Novikova et al. 2006). From this information, ADVANTG constructs a discretized representation of the transport problem on a user-provided spatial grid and given multigroup cross section library. ADVANTG can automatically launch Denovo to execute the discrete ordinates calculation, possibly using multiple cores and/or processors (e.g., on multi-core desktop systems and clusters). The discretized models and discrete ordinates solutions can be visualized using the open-source VisIt 3-D visualization software (Childs et al. 2005).

The primary objective of ADVANTG is to reduce both the user effort and the computational time required to obtain accurate and precise tally estimates across a broad range of challenging transport applications. ADVANTG has been applied to simulations of real-world radiation shielding, detection, and neutron activation problems. Examples of shielding applications include material damage and dose rate analyses of the Oak Ridge National Laboratory (ORNL) Spallation Neutron Source and High Flux Isotope Reactor (Risner and Blakeman 2013) and the ITER Tokamak (Ibrahim et al. 2011). ADVANTG has been applied to a suite of radiation detection, safeguards, and special nuclear material movement detection test problems (Shaver et al. 2011). ADVANTG has also been used in the prediction of activation rates within light water reactor facilities (Pantelias and Mosher 2013). In these projects, ADVANTG was demonstrated to significantly increase the tally figure of merit (FOM) relative to an analog MCNP simulation. The ADVANTG-generated parameters were also shown to be more effective than manually generated geometry splitting parameters.

ADVANTG provides a powerful, efficient, and fully automated alternative to traditional methods for generating variance reduction parameters. Because ADVANTG employs a deterministic transport solver, no extra effort is required on the part of the user to generate weight-window parameters that span the entire problem domain. For challenging problems, multiple iterations may be required to refine the deterministic spatial mesh, quadrature set, or other computational options to obtain high-quality variance reduction parameters. This process can generally be accomplished in much less time overall and with less effort than using a stochastic weight-window generator or manually developing cell importances.

The variance reduction generator methods implemented in ADVANTG are described in Section 2. Implementation of the methods is summarized in Section 3. Running ADVANTG from the command line is the subject of Section 4. Input and output are described in Sections 5 and 6, respectively. Several example problems are described in detail in Section 7. Tips and suggestions for using ADVANTG to obtain accurate and precise MCNP tally estimates are discussed in Section 8. Finally, known limitations are listed in Section 9.

2. METHODS

ADVANTG implements the Consistent Adjoint Driven Importance Sampling (CADIS) method (Wagner and Haghghat 1998) and the Forward-Weighted CADIS (FW-CADIS) method (Wagner et al. 2014) for generating variance reduction parameters. The CADIS and FW-CADIS methods provide a prescription for generating space- and energy-dependent weight-window targets and a consistent biased source distribution. The CADIS method was developed for accelerating individual tallies, whereas FW-CADIS was developed for multiple tallies and mesh tallies. The CADIS method has been demonstrated to provide speed-ups in the tally FOM of $O(10^1-10^4)$ across a broad range of radiation detection and shielding problems. The FW-CADIS method has been shown to produce relatively uniform statistical uncertainties across multiple cell tallies and large space- and energy-dependent mesh tallies in real-world applications (Wagner et al. 2010).

2.1 CADIS METHODOLOGY

The CADIS method was developed for transport problems in which a single scalar quantity is to be estimated. Consider the fixed-source transport equation

$$\mathbf{H}\psi(\vec{\mathbf{r}}, \hat{\boldsymbol{\Omega}}, E) = q, \quad (2-1)$$

where \mathbf{H} is the transport operator, q is the known source distribution, ψ is the unknown angular flux density, and boundary conditions are given. It is assumed that the quantity of interest can be written as the integral

$$R = \langle \sigma_d, \psi \rangle, \quad (2-2)$$

where the angle brackets denote integration over all phase-space variables. In Eq. (2-2), σ_d denotes an arbitrary response function, for example, a detector cross section or flux-to-dose-rate conversion factor.

Associated with Eqs. (2-1) and (2-2) is the adjoint transport equation

$$\mathbf{H}^+\psi^+ = \sigma_d, \quad (2-3)$$

where \mathbf{H}^+ is the adjoint transport operator and ψ^+ is the adjoint flux density. The adjoint transport operator is related to the forward operator by

$$\langle \psi, \mathbf{H}^+\psi^+ \rangle = \langle \psi^+, \mathbf{H}\psi \rangle, \quad (2-4)$$

and thus the response can also be written in terms of the adjoint flux

$$R = \langle q, \psi^+ \rangle. \quad (2-5)$$

The boundary conditions of Eq. (2-3) are chosen to be identical to the forward conditions, though they apply to the opposite directional half-space (i.e., to outgoing as opposed to incoming directions).

The solution of Eq. (2-3) can be interpreted as an importance function (Bell and Glasstone 1970). This can be understood by setting $q = \delta(\mathbf{P} - \mathbf{P}_0)$, where δ is the Dirac delta function and $\mathbf{P}_0 = (\vec{\mathbf{r}}_0, \hat{\boldsymbol{\Omega}}_0, E_0)$ denotes an arbitrary point in the problem phase-space. In this case, Eq. (2-4) reduces to

$$\psi^+(\mathbf{P}_0) = \int G(\mathbf{P}_0 \rightarrow \mathbf{P})\sigma_d(\mathbf{P})d\mathbf{P}, \quad (2-6)$$

where the forward solution is the Green's function G . This equation states that $\psi^+(\mathbf{P}_0)$ is the expected contribution to the response R from a unit-weight particle emitted at \mathbf{P}_0 . This property makes the adjoint function particularly useful in Monte Carlo simulations; it can be used to determine whether a particle's trajectory will carry it toward a region where it is relatively more or less likely to contribute to the tally of interest. For this reason, the solution of the adjoint transport equation is often referred to as an *importance function* or *importance map*.

The CADIS method is a recipe for calculating space- and energy-dependent weight-window targets and a consistent biased source distribution using an estimate of the adjoint function. (For a description of the weight-window variance reduction technique, see, for example, the MCNP Manual, Vol. I, Sec. 2.VII.B.6.) First, an importance map is generated by solving Eq. (2-3) and appropriate boundary conditions using a relatively inexpensive deterministic transport calculation. Weight targets are then computed in proportion to the inverse of the adjoint scalar flux

$$w(\mathbf{P}) = \frac{R}{\phi^+(\mathbf{P})}. \quad (2-7)$$

For the MCNP code, weight-window lower bounds are generated as

$$w_\ell(\mathbf{P}) = \frac{2}{(1+r)} \frac{R}{\phi^+(\mathbf{P})}, \quad (2-8)$$

where r is the ratio of the upper and lower weight bounds. A unique feature of the CADIS method is the use of a biased source distribution

$$\hat{q}(\mathbf{P}) = \frac{q(\mathbf{P})\phi^+(\mathbf{P})}{R}, \quad (2-9)$$

which ensures that source particles are preferentially sampled in regions of high importance. In addition, each source particle will start with a weight consistent with Eq. (2-7).

The variance reduction parameters in Eqs. (2-7) and (2-9) depend on the response value, R , which is the quantity to be estimated. If highly accurate response and adjoint flux estimates were required to produce useful variance reduction parameters, then this approach would be of little value. Fortunately, experience has shown that this is not the case. For some problems, even crude estimates can be used to generate effective variance reduction parameters.

2.2 MULTIPLE TALLIES

The CADIS method is an effective technique for estimating a single quantity of interest. In many problems, though, one desires to estimate multiple quantities. Consider a simulation in which estimates are sought for multiple responses with uniform statistical precision:

$$R_i = \langle \sigma_{d,i}, \phi \rangle, \text{ for } i = 1, 2, \dots, N, \quad (2-10)$$

where both the $\sigma_{d,i}$ and N are arbitrary. Several approaches for generating variance reduction parameters that seek to satisfy this objective are described in the sections that follow.

2.2.1 CADIS Method

In some cases, the CADIS method can be effectively applied to estimate multiple tallies. A straightforward application of CADIS to a simulation with N different tallies would be to calculate N different adjoint solutions, generate N different sets of variance reduction parameters, and execute N different Monte Carlo simulations. This approach can be reasonable when N is small. For tallies with many cell or energy bins or for mesh tallies, though, this technique is generally inefficient.

A second approach would be to simply treat the sum of the responses as the response of interest in Eq. (2-2):

$$R = R_1 + R_2 + \cdots + R_N, \quad (2-11)$$

so that

$$q^+ = \sigma_{d,1} + \sigma_{d,2} + \cdots + \sigma_{d,N} = \sigma_d. \quad (2-12)$$

This technique can be very effective, for example, in problems where the tallies reside within the same vicinity of the problem domain. However, when it is applied to tallies located at significantly different distances from the source, the tally FOMs will generally differ greatly in magnitude. In many cases, the tally farthest from the source will have an FOM on par with an analog simulation. This is a consequence of the CADIS method's definition of importance as the expected contribution to the total response R . Relatively fewer contributions are made to tallies with relatively lower expected values.

A third technique can be effective when estimates are sought for tallies over concentric regions surrounding the source. In this case, defining the response of interest to be the tally in the outermost region will generally reduce the variance of all of the tallies. Of course, this occurs simply because particles must pass through the inner tally volumes to reach the outermost region.

In most cases, application of the CADIS method to simultaneously estimate multiple tallies will produce an undesirable amount of variation in tally FOMs. For these problems, a different approach to constructing the adjoint source is needed.

2.2.2 Cooper and Larsen Method

Cooper and Larsen (2001) developed a method for constructing weight windows for global transport problems, in which flux estimates are sought across the entire spatial domain. The authors showed that the flux density of Monte Carlo particles is related to the physical flux by

$$m(\mathbf{P}) = \frac{\phi(\mathbf{P})}{\bar{w}(\mathbf{P})}, \quad (2-13)$$

where \bar{w} is the mean particle weight at phase-space location \mathbf{P} . Thus, if the centers of the weight windows are chosen to be proportional to the forward flux, then an approximately uniform Monte Carlo particle flux (i.e., computational particle flux) will be obtained throughout the problem. While obtaining uniform Monte Carlo flux density is not exactly equivalent to obtaining uniform statistical uncertainties, the authors demonstrated that this choice for the weight window produced nearly uniform relative variances in numerical tests.

2.3 FW-CADIS METHOD

The FW-CADIS method was developed with an objective similar to that of the Cooper and Larsen method—that is, to generate variance reduction parameters for simultaneous estimation of multiple tallies with approximately uniform statistical precision. However, whereas Cooper and Larsen’s method was designed for global problems, the FW-CADIS method is intended to span the range from a few localized tallies to space- and energy-dependent mesh tallies that encompass the entire domain. This is accomplished by constructing an adjoint source that consists of appropriately weighted contributions from all tallies of interest. The weights are the inverses of the individual responses:

$$q^+ = \frac{1}{R_1} \sigma_{d,1} + \frac{1}{R_2} \sigma_{d,2} + \cdots + \frac{1}{R_N} \sigma_{d,N}. \quad (2-14)$$

Then the total response is a sum of equal-weight terms

$$R = \langle q^+, \phi \rangle = 1 + 1 + \cdots + 1. \quad (2-15)$$

Because the CADIS method defines importance as the expected contribution to the total response R , approximately the same number of contributions will be made to all tallies regardless of their expected values. Implicit in this argument is the assumption that every particle contributes to just a single tally. Though this assumption is often not strictly valid, relatively uniform uncertainties are obtained in most problems.

To construct the weighted adjoint source in Eq. (2-14), estimates of individual responses (R_i , $i = 1, 2, \dots, N$) are required. For this reason, applying the FW-CADIS method requires two deterministic calculations: an initial forward calculation to estimate the responses and an adjoint calculation to estimate the importance function resulting from the weighted adjoint source. The importance function is then used to construct weight-window bounds and a biased source distribution according to Eqs. (2-7) and (2-9), respectively. In essence, FW-CADIS is a recipe for constructing an adjoint source that can be used within the CADIS framework.

In the subsections that follow, Eq. (2-14) is used to derive adjoint sources for the common cases of interest. Spatial weighting options will be developed for problems with multiple cell-averaged tallies and mesh tallies. Two energy weighting options are also developed for estimating energy-integrated tallies or detailed energy spectra. These options can be used in different combinations to tailor the biasing parameters for a particular calculation.

2.3.1 Path-Length Weighting

Consider a problem in which estimates are sought for an arbitrary number of cell-averaged responses with approximately uniform statistical precision. Let the volume of the i^{th} cell be denoted by V_i . The i^{th} response to be estimated is then

$$R_i = \frac{1}{V_i} \int \sigma_i(E) \int_{V_i} \phi(\vec{r}, E) dV dE, \quad (2-16)$$

where σ_i is a tally multiplier—for example, an energy-dependent cross section, a flux-to-dose-rate conversion function, or just a constant. Using Eq. (2-10), we find that

$$\sigma_{d,i}(\vec{\mathbf{r}}, E) = \frac{1}{V_i} f_i(\vec{\mathbf{r}}) \sigma_i(E), \quad (2-17)$$

where f_i is the indicator function:

$$f_i(\vec{\mathbf{r}}) = \begin{cases} 1, & \text{for } \vec{\mathbf{r}} \in V_i, \\ 0, & \text{otherwise.} \end{cases} \quad (2-18)$$

Now using Eq. (2-14) we find that the adjoint source for the tally in the i^{th} cell is

$$q_i^+(\vec{\mathbf{r}}, E) = \frac{f_i(\vec{\mathbf{r}}) \sigma_i(E)}{\int \sigma_i(E') \int_{V_i} \phi(\vec{\mathbf{r}}, E') dV dE'}. \quad (2-19)$$

As expected, the adjoint source density increases as the flux in the cell or the cell volume decreases. Moreover, the magnitude of σ_i has no impact on the importance function. This is appropriate, because all cell tallies in MCNP are track-length based, so tally multipliers (e.g., from an FM card) do not contribute to variance.

The spatial weighting in Eq. (2-19) is referred to as *path-length weighting* and is the default treatment in ADVANTG. It can be used with cell and surface tallies. (Note that surface tallies have an associated volume after being mapped onto the deterministic mesh.) It can also be used with mesh tallies, however the global weighting technique (described in the next subsection) is generally preferred. When path-length weighting is applied to mesh tallies, statistical uncertainties will generally be lowest/highest in mesh tally voxels that contribute the most/least to the volume-averaged response.

2.3.2 Global Weighting

Consider a problem in which the desire is to obtain approximately uniform statistical precision across a large tally volume (e.g., a mesh tally). Let the tally volume, V , be subdivided into N regions, each having volume $\Delta V = V/N$. If path-length weighting is applied to each of the regions using Eq. (2-19), then

$$q^+(\vec{\mathbf{r}}, E) = \sum_{i=1}^N \frac{f_i(\vec{\mathbf{r}}) \sigma_i(E)}{\int \sigma(E') \int_{\Delta V} \phi(\vec{\mathbf{r}}, E') dV dE'}. \quad (2-20)$$

Because CADIS parameters are derived from the ratio $\phi^+ / \langle \phi, q^+ \rangle$, multiplying the adjoint source by a constant has no effect on the final variance reduction parameters. If we divide the denominator of Eq. (2-20) by ΔV and then consider the limiting case for large N , we arrive at

$$q^+(\vec{\mathbf{r}}, E) = \frac{f(\vec{\mathbf{r}}) \sigma(E)}{\int \sigma(E') \phi(\vec{\mathbf{r}}, E') dE'}, \quad (2-21)$$

where $f(\vec{\mathbf{r}})$ is the indicator function for the mesh tally volume.

The spatial treatment in Eq. (2-21) is referred to as *global weighting*, though it can be applied to a mesh tally of any size. It must be explicitly turned on (see the description of the `fwcadis_spatial_treatment` card in Section 5.4.1). Because the adjoint source was developed based on equal-volume subdivisions of the mesh tally region, only the outer boundary of the tally region is considered when constructing the adjoint source (i.e., the locations of the interior mesh planes are ignored). Smaller than average voxels will tend to have larger than average statistical uncertainties, and vice versa.

2.3.3 Response Weighting

In the previous two subsections, energy-integrated tallies are considered. As a result, the adjoint sources shown in Eqs. (2-19) and (2-21) are normalized by the energy-integrated response. For historical reasons, this type of energy weighting is referred to as *response weighting*. This normalization is appropriate, for example, when estimating total fluxes, dose rates, and reaction rates. It can be used regardless of whether the tally region is a point, surface, or volume.

With response weighting, tally statistical uncertainties will generally be lowest at energies that contribute most strongly to the total response. Estimating energy-dependent tallies with approximately uniform precision across all energy bins is possible (as described in the next subsection), but is generally more computationally expensive than response weighting and is needed less often. For this reason, response weighting is the default energy treatment in ADVANTG.

2.3.4 Spectral Tallies

In problems where detailed spectral information is desired, response weighting can be turned off (see the `fwcadis_response_weighting` input keyword in Section 5.4.1). In the case of path-length weighting, the adjoint source becomes:

$$q_i^+(\vec{\mathbf{r}}, E) = \frac{f_i(\vec{\mathbf{r}})}{\int_{V_i} \phi(\vec{\mathbf{r}}, E) dV}. \quad (2-22)$$

For global weighting, the adjoint source is

$$q^+(\vec{\mathbf{r}}, E) = \frac{f(\vec{\mathbf{r}})}{\phi(\vec{\mathbf{r}}, E)}. \quad (2-23)$$

At a given point in space, the magnitudes of the adjoint sources shown above are relatively higher at energies where the flux is relatively lower in magnitude. In this way, energy-dependent tallies can be estimated with approximately uniform precision across all energy bins. This treatment generally results in a significant increase in the average number of splitting events per history, and thus an increase in computational time, relative to response weighting.

3. IMPLEMENTATION

This section discusses implementation of the methods described in Section 2. The objective here is not to provide a thorough and complete description of all algorithms implemented in ADVANTG. Instead, the basic operation of ADVANTG is discussed, along with the details of algorithms that affect the accuracy of the model discretization process and, by extension, the quality of the deterministic results and variance reduction parameters.

3.1 COMPUTATIONAL SEQUENCES

ADVANTG performs a series of computational steps to implement the CADIS and FW-CADIS methods. The steps are listed and briefly described in Table 3-2. Only certain steps are included in the execution of each method, as shown in Table 3-3. For future reference, the second column of Table 3-3 lists the input option that selects each sequence (see the description of the method input keyword in Section 5.2). ADVANTG provides a third sequence, shown in the last row of Table 3-3, which discretizes the problem geometry, source, and tallies, and outputs the discretized model for visualization and inspection purposes. This dx sequence can also be used to run a forward or adjoint Denovo discrete ordinates calculation without generating variance reduction parameters (see Section 5.4.2).

3.2 MULTIGROUP CROSS SECTION LIBRARIES

The discrete ordinates calculations performed by Denovo require multigroup cross sections. The ADVANTG distribution includes several ANISN-format coupled neutron-gamma cross section libraries, listed in Table 3-1. For future reference, the second column of the table lists the `anisn_library` input option used to select each library (see Section 5.6). The library energy group structures are given in Appendix A. No auxiliary codes are needed to use these libraries. ADVANTG can read ANISN-format libraries, mix cross sections, and output a working library for Denovo.

Table 3-1. Multigroup libraries

Library	<code>anisn_library</code> option	# of groups (N / G)	# of isotopes or elements	Evaluation	Reference
27n19g	27n19g	27 / 19	393	ENDF/B-VII.0	Wiarda et al. 2008
200n47g	200n47g	200 / 47	393	ENDF/B-VII.0	Wiarda et al. 2008
BUGLE-96	bugle96	47 / 20	120	ENDF/B-VI.3	White et al. 1995
BPLUS	bplus	47 / 20	393	ENDF/B-VII.0	N/A
DABL69	dabl69	46 / 23	80	ENDF/B-V	Ingersoll et al. 1989
DPLUS	dplus	46 / 23	393	ENDF/B-VII.0	N/A
FENDL67	fendl67	46 / 21	71	FENDL-2.1	López Aldama and Trkov, 2004

Table 3-2. Computational steps

Step	Tasks
A	<ul style="list-style-type: none"> • Read and check user input • Generate and read an MCNP runtime file • Mix multigroup cross sections • Map material regions onto the deterministic spatial mesh
B	<ul style="list-style-type: none"> • Map tally regions onto mesh
C	<ul style="list-style-type: none"> • Map MCNP SDEF source onto mesh and energy groups
D	<ul style="list-style-type: none"> • Setup and execute forward Denovo calculation • Read forward Denovo flux solution
E	<ul style="list-style-type: none"> • Generate FW-CADIS adjoint source
F	<ul style="list-style-type: none"> • Generate CADIS adjoint sources from tallies
G	<ul style="list-style-type: none"> • Setup and execute adjoint Denovo calculation • Read adjoint Denovo flux solution
H	<ul style="list-style-type: none"> • Generate and write weight-window bounds • Estimate biased source probabilities • Write new MCNP input file with WWP and SB cards
I	<ul style="list-style-type: none"> • Write Silo output for visualization

Table 3-3. Computational sequences

Method or sequence	method option	Steps included								
		× = always included					• = optional			
		A	B	C	D	E	F	G	H	I
CADIS	<code>cadis</code>	×	×				×	×	×	•
FW-CADIS	<code>fwcadis</code>	×	×	×	×	×		×	×	•
Discretize-only or Denovo-only calculation	<code>dx</code>	×	•	•	•		•	•		•

The 27n19g and 200n47g libraries are general-purpose shielding libraries based on a weighting function that consists of a fission spectrum, a $1/E$ slowing down spectrum, and a Maxwellian distribution. The BUGLE-96 library was developed for light water reactor shielding and pressure vessel dosimetry applications. The broad-group cross sections were generated by collapsing the VITAMIN-B6 library using five different weighting spectra calculated from a 1-D model of a reactor cavity and bioshield. The DABL69 library was developed for use in defense-related radiation shielding applications. It was created by collapsing the VITAMIN-E library using a weighting function similar to the 200n47g library, but with an added 14 MeV fusion peak. The FENDL67 library was developed for fusion applications by collapsing the FENDL/MG-2.1 library (175 neutron and 42 gamma groups) to a broad-group structure.

The BPLUS and DPLUS libraries, developed by the ADVANTG team, are updated versions of the BUGLE-96 and DABL69 libraries, respectively. These libraries were generated using the same group structures and weighting spectra as their older counterparts, but they include all 393 isotopes in the ENDF-B/VII.0 evaluation data files. The BPLUS and DPLUS libraries have not yet been thoroughly validated, but they have been used to generate effective variance reduction parameters for many problems.

3.3 MATERIAL COMPOSITION MAPPING

For discrete ordinates calculations, a multigroup cross section working library must be generated based on the material compositions defined on the *m* cards in the MCNP input file. This task requires mapping MCNP nuclide identification numbers (ZAIDs) (e.g., 26056 for ^{56}Fe) to ANISN cross section table IDs. For this purpose, each of the multigroup libraries distributed with ADVANTG has an associated ZAID-to-index mapping file (*.zaid* file) that defines the default mapping. Users can override and/or add mappings using the *anisn_zaid_map* keyword (see Section 5.6).

The ZAID-to-index mapping process can require one or two steps. The ZAID is first located in the mapping database and, if an associated ANISN table exists, it is used immediately. If the ZAID was not found in the database, the next step will depend on the type of multigroup library being used:

- If the library is a coupled neutron-photon or neutron-only library and the ZAID refers to an element (e.g., 26000), then the element is expanded into its naturally occurring isotopes based on the abundances listed in Rosman and Taylor (1998). A search is then performed to find mappings for all of the isotopes. If the total abundance of isotopes that are not found in the database is less than 0.5%, then the abundances for the remaining isotopes are renormalized and used. If the fraction of missing isotopes is greater than 0.5%, the expansion process is aborted, and an error is generated.
- If the library is a photon-only library, then a search is made to find a ZAID with the same *Z* number (proton number). If one is found, its mapping is used. Otherwise, an error is generated.

In all other cases, an error message is generated. For convenience, ADVANTG attempts to find multigroup cross section tables for all materials before issuing an error message. If a mapping error does occur, all of the missing ZAIDs are listed in the message.

3.4 MATERIAL REGION MAPPING

The Denovo discrete ordinates code solves a discretized form of the transport equation on a structured rectangular mesh. The MCNP5 code uses a combinatorial geometry in which material cells are described as volumes bounded by several possible types of surfaces (planes, cylinders, spheres, cones, etc.). A

fundamental task in creating an approximate representation of the Monte Carlo model for Denovo is to map the combinatorial geometry onto a user-specified structured grid.

To illustrate the material mapping process implemented in ADVANTG, consider test problem INP12 from McKinney and Iverson (1996), shown in Fig. 3-1. This problem models an oil-well logging scenario in which an iron sonde containing two ^3He detectors and a neutron source is inserted into a water-filled borehole within a limestone formation. (The geometry of the problem is divided into mesh-like cells to utilize cell-based weight-window parameters from a diffusion code. The input file for this problem was created before mesh-based weight windows were available in MCNP.) For demonstration purposes, the original point source defined in the problem was changed to a volume source that fills the source cell.

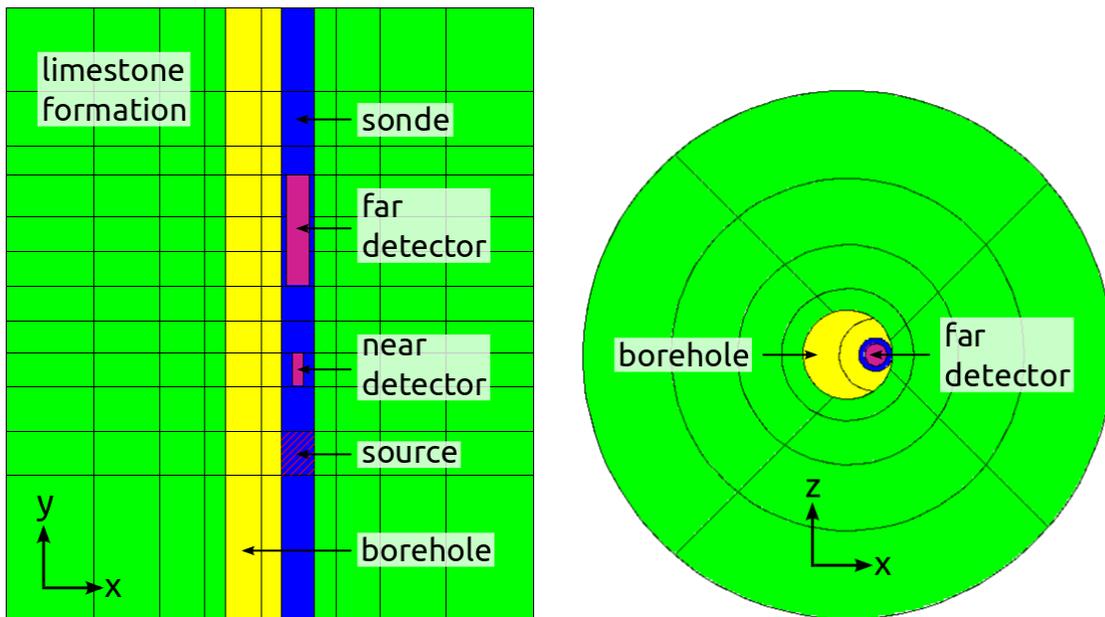


Fig. 3-1. Geometry of problem INP12 with modified source.

ADVANTG performs material region mapping by tracing rays through the MCNP geometry model. The starting points of rays are randomly sampled on the exterior $-x$, $-y$, and $-z$ faces of the mesh. Rays are then traced in the $+x$, $+y$, and $+z$ directions to the opposite side of the mesh from the starting location. As each ray is traced, tallies record the track length each ray traverses through each material within each voxel of the structured grid. This process is illustrated in Fig. 3-2, which depicts two rays that started from the same external voxel face and were traced in the $+x$ direction through the borehole region of problem INP12. In the figure, the gray dashed lines represent mesh boundaries, and the blue crosses denote locations where the rays cross a mesh plane, and the red crosses indicate where the rays cross an MCNP cell surface. In the center voxel, for example, the ray with greater z -coordinate contributes three track-length scores for three different materials, whereas the other ray contributes only a single score.

Once the ray tracing is completed, the track-length tallies are used to estimate the volume fractions of materials within each voxel. The material fractions are then used to generate a mixed-material specification for the Denovo calculation. Materials with volume fractions that differ by less than a tolerance value are combined to minimize the total number of mixed materials generated. The tolerance

can be decreased to obtain more accurate material maps in problems where the response is sensitive to the volume of one or more materials.

The material maps generated by ADVANTG can be displayed using the VisIt visualization software. The material map generated for problem INP12 on an $81 \times 57 \times 81$ voxel nonuniform mesh is shown in Fig. 3-3. In the figure, two images are displayed, both of which are clipped at $z = 0$ to show the geometry at the center of the borehole region. In the left image, distinct material interfaces were reconstructed by VisIt before displaying the plot. (VisIt applies this treatment by default.) The right image uses a *clean zones only* option, which displays voxels with mixed materials as white. No interface reconstruction is performed in either ADVANTG or Denovo, so it is important to consider the impact that material mixing will have on the discrete ordinates calculation. For this reason, examining both types of material plots is recommended.

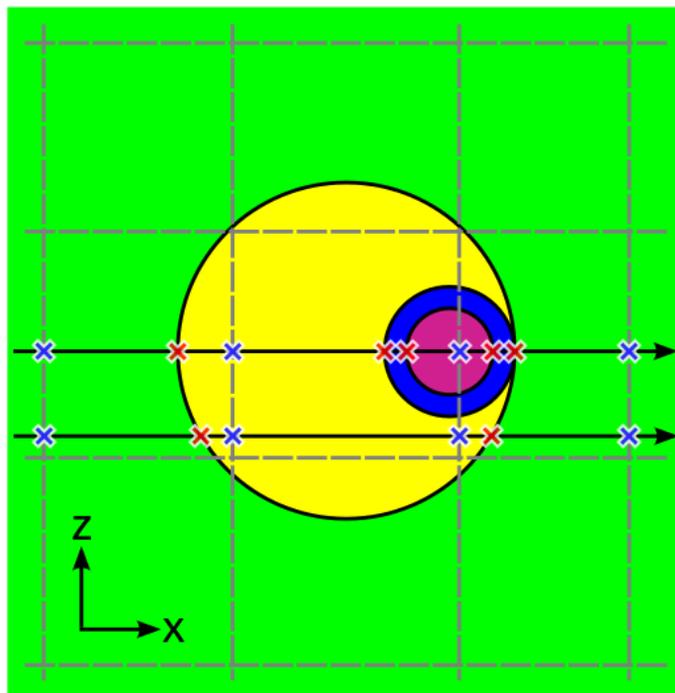


Fig. 3-2. Ray tracing in the $+x$ direction through problem INP12.

By default, ADVANTG will trace a minimum of ten rays from each external voxel face on the $-x$, $-y$, and $-z$ edges of the mesh. Increasing the number of rays will produce more accurate material volume fractions and thus a more accurate deterministic calculation. (See Section 5.3.1 for a description of the ray tracer settings.) The computational time consumed by the ray tracing is proportional to the number of rays. The average cost of tracing a single ray can vary greatly between MCNP models. Ray tracing is generally more expensive in models that contain many complicated cells (i.e., cells that are defined using the union or complement operators).

Denovo supports specularly reflective boundary conditions, but only on boundaries that are perpendicular to one of the coordinate axes. ADVANTG provides the capability to unfold reflected geometries that cannot be modeled directly by Denovo (see Section 5.3.1 for the associated input options). For example, many MCNP models of the ITER Tokamak consist of a sector of the reactor with reflective boundary

conditions on the external azimuthal surfaces. Fig. 3-4 shows the discretized geometries generated for a 40° sector ITER model with and without the unfolding option (Ibrahim et al. 2011). Note that for certain geometries, it may be necessary to extend the boundaries of the model in order for the unfolding to work properly. In all cases, careful inspection of the unfolded geometry in VisIt is highly recommended.

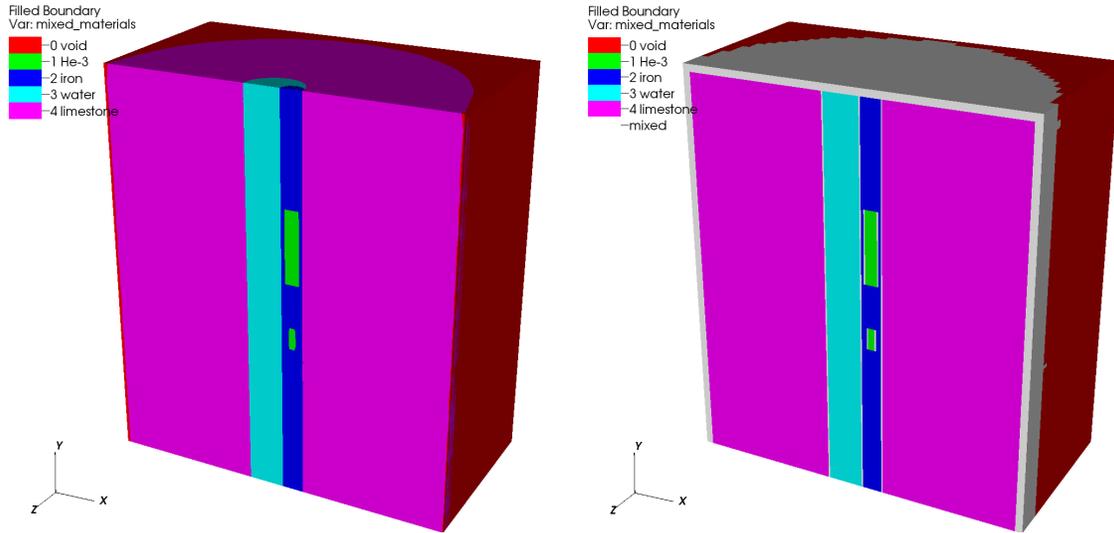


Fig. 3-3. Mixed material distribution of problem INP12. The left image was generated using the default material interface reconstruction technique, and the right image was generated using *clean zones only* option.

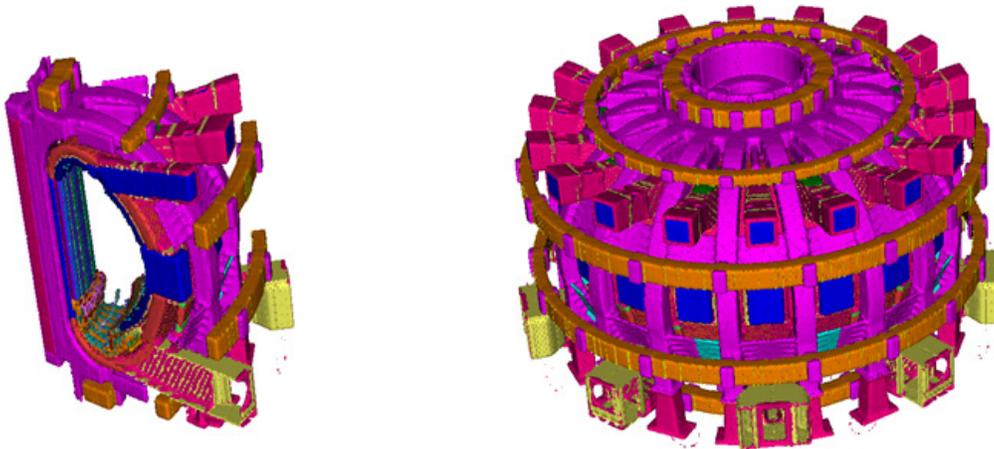


Fig. 3-4. ITER 40° sector model (left) and unfolded model (right).

3.5 SOURCE MAPPING

When setting up a forward discrete ordinates calculation, ADVANTG maps the fixed source defined in an MCNP input (via the SDEF card) into a form acceptable to Denovo. The mapped source consists of the following data:

- List of multigroup energy spectra
- List of any point sources, including the location, strength, and spectrum index
- Array of volume source intensities for each mesh voxel
- Array of spectrum indices for each mesh voxel

This source specification may contain point sources, volume sources, or both. Also, the same spectrum can be used for multiple points or voxels.

Point sources are treated differently than volume sources in Denovo. This is because localized sources tend to cause ray effects in discrete ordinates solutions, especially when the source is located within an optically thin or strongly absorbing medium. Ray effects due to localized sources can be mitigated, and in many cases nearly eliminated, by replacing point and small volume sources with the corresponding first-collided scattering source, which is distributed throughout the problem domain. For point sources, Denovo automatically generates an energy-dependent first-collided source based on an uncollided flux distribution that it calculates using an analytic point-kernel approach. The total flux is then calculated as the sum of the uncollided flux and the converged S_N solution for the first-collided source. By default, Denovo does not apply a first-collision treatment to volume sources; however, a Monte Carlo-based approach to estimating the first-collision source for a volume source can be activated if desired. (See Section 5.7 for a description of Denovo options.)

ADVANTG provides an option to treat a volume source as a point source in the discrete ordinates calculations, and it is generally recommended to do so whenever the source is very small relative to the size of the problem domain. The location of the source point is determined by sampling the source and averaging the sampled positions. If the source is uniform in volume, the point source will be located at the centroid of the source region with some offset due to sampling error. (See Section 5.3.1 for a description of the MCNP source settings.)

In MCNP, SDEF sources can be specified so as to contain a mix of volume and point sources. For simplicity, ADVANTG treats an SDEF source as either a volume source or a collection of one or more point sources. Though there are several ways of constructing a distribution of point sources using the SDEF cards, ADVANTG will detect only the following cases:

- If none of the POS, X, Y, Z, RAD, or EXT variables appear on the SDEF card, the source is treated as a point source located at the origin, as in MCNP.
- If neither the RAD nor EXT variables appears on the SDEF card and the X, Y, and Z variables are given as explicit or default values, the source is treated as a point source located at (X, Y, Z).
- If POS is an explicit value or an L, S, DS L, or DS S distribution and none of the X, Y, Z, RAD, or EXT variables is listed on the SDEF card, the source is treated as the collection of points defined in the POS distribution or in its child distributions.

In all other cases, ADVANTG will treat the source as a volume source.

ADVANTG passes the locations of point sources directly to Denovo. Volume sources, on the other hand, must first be mapped onto the voxels of the spatial mesh. This is done by sampling source particles and then tallying the particle weight in the voxel in which the source point is located. The spatial distribution of the source density is then estimated as the average weight in each voxel. In this mapping process, the number of source samples to be simulated is determined based on three user input parameters: the minimum and maximum number of samples and a target minimum average number of samples per mesh voxel containing source. The intent of the target is to provide a relatively problem-independent means for obtaining consistently sampled volume sources. Because the spatial extent of the source is not known *a priori*, the number of samples is determined during sampling. Initially, the minimum number of samples is drawn. The number of samples is then divided by the number of voxels that have been found to contain source. If this number is greater than or equal to the target density, no additional samples are taken. Otherwise, another sample is drawn, and the stopping criterion is re-evaluated. Sampling proceeds until either the target is met or the maximum number of samples is reached, in which case a warning message is printed. The spatial distribution of volume sources generated by ADVANTG can be displayed in VisIt. The source region generated for problem INP12 is shown in Fig. 3-5.

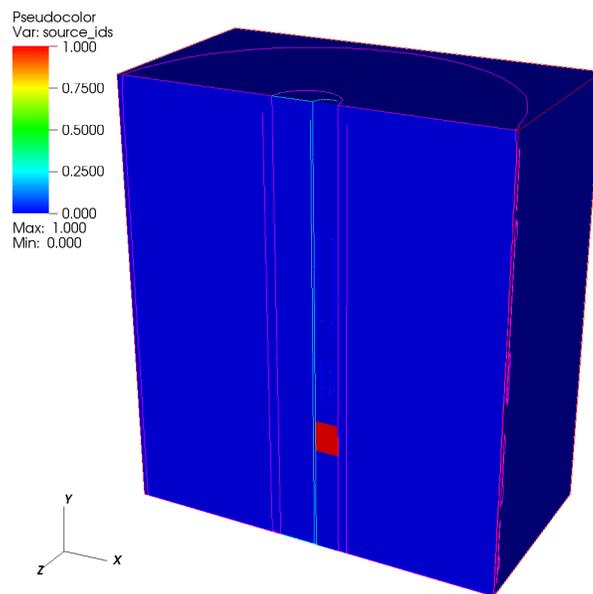


Fig. 3-5. Source regions in problem INP12.

ADVANTG maps SDEF energy distributions onto the group structure of the multigroup cross section library using analytic integration. All distribution types are supported: discrete lines (SI L), histograms (SI H), piecewise-linear densities (SI A), built-in functions (SP -f), as well as distributions of arbitrary sets of distributions (SI S). If more than one energy distribution is defined and the source is separable in space and energy, ADVANTG collapses the distributions into a single spectrum. If the source is not separable, a multigroup spectrum is generated for each distribution and then mapped to spatial elements later. Though ADVANTG is capable of mapping all types of distributions, the use of the DS H, DS L, and DS T cards for the ERG variable is not currently supported because this would require partitioning the energy distribution bins in a spatially dependent manner. For the same reason, if ERG is an independent variable that is depended upon by another variable, it must be specified as an S distribution.

If the source is separable in space and energy, each source point or voxel will have the same spectrum. If the source is not separable, the association between points or voxels and spectra must be determined. For point sources, this is done by looking at the dependencies between distributions. For volume sources, the number of samples that have appeared from each energy distribution is tracked for each voxel. If a point or voxel is found to contain a mix of spectra, the already-mapped spectra are combined based on their relative contributions, as determined from the probabilities defined in the SDEF distributions (for point sources) or based on the relative number of observed samples (for volume sources).

3.6 TALLY REGION MAPPING

To construct adjoint source distributions for the Denovo S_N calculations, ADVANTG maps surface, cell, and mesh tallies onto the user-specified spatial grid. Tally region mapping is done simultaneously with material region mapping to avoid ray-tracing through the geometry a second time. The discretized tally regions generated by ADVANTG can be displayed in VisIt, as shown in Fig. 3-6 for problem INP12. In the figure, the tally IDs correspond to the order in which they are listed in the ADVANTG input file. In this example, the near detector was listed before the far detector. ADVANTG supports cell tallies with multiple cell bins and tallies on repeated structures and lattices. Tally regions may be missed by the ray tracer if they are small relative to the mesh voxels in which they reside. If this occurs, the mesh should be refined and/or the number of rays should be increased.

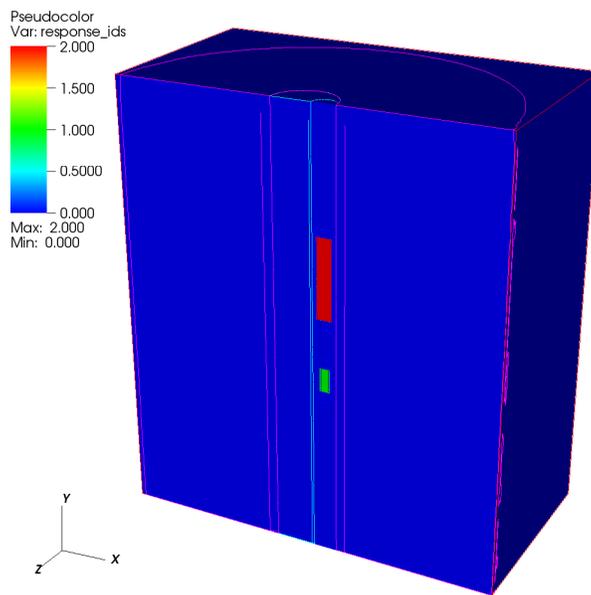


Fig. 3-6. Response regions in problem INP12.

From the perspective of the CADIS and FW-CADIS methods, all tallies have an associated energy spectrum. For example, the DE/DF cards define the response spectrum for tallies that have associated dose functions. As with forward source spectra, ADVANTG maps the response spectrum onto the multigroup structure using analytic integration. ADVANTG supports DE/DF, FM, and EM multipliers. Though MCNP allows a tally to have more than one type of multiplier, ADVANTG will select and use only a single type of multiplier. The order of precedence is DE/DF, FM, then EM. For example, if a tally has DE/DF cards, any FM or EM cards will be ignored. If no multiplier is defined, a uniform spectrum is used. For pulse-height

(F8) tallies, ADVANTG uses the total cross section of the material in the tally cell as the response spectrum. This treatment has been found to be effective for most problems involving pulse-height tallies.

3.7 DETERMINISTIC CALCULATIONS

ADVANTG obtains deterministic transport solutions by preparing inputs for and executing the Denovo 3-D, block-parallel discrete ordinates package. Denovo was selected for this purpose because it provides a powerful, robust, and efficient general deterministic transport capability. It is built around modern linear algebra solvers provided by the Trilinos library (Heroux et al. 2003) and uses the Generalized Minimum RESidual (GMRES) solver to converge within-group (scattering) iterations. This Krylov subspace solver has been shown to significantly outperform source iteration (Richardson iteration) in problems with thick scattering media. Denovo also implements several spatial discretization schemes, quadrature sets, and both analytic point-kernel and Monte Carlo-based first collision sources. For large-scale problems, the transport sweeps can be performed in parallel on multiple processors. Denovo has been used to solve several problems with more than one billion spatial cells.

Denovo provides multiple front ends for input. The primary and full-featured front end is implemented as a Python (scripting language) module. ADVANTG drives Denovo through this Python interface, which provides access to multiple spatial discretization schemes (step characteristics, linear discontinuous, trilinear discontinuous, and weighted diamond difference), multiple quadrature sets (quadruple range, LDFE, Gauss-Legendre product, and level symmetric), and the ability to perform parallel calculations.

Both group-wise and energy-integrated scalar flux solutions generated by Denovo can be displayed in VisIt. Fig. 3-7 shows the discretized material map (left) and the energy-integrated adjoint scalar flux (right) from a CADIS calculation where the far detector was the response of interest. The clip operator was used to arrange the material and flux plots next to each other. Contours were also plotted and overlaid on the pseudocolor flux plot. The near detector does not show up in the material map because the CADIS calculation was performed on a relatively coarse mesh.

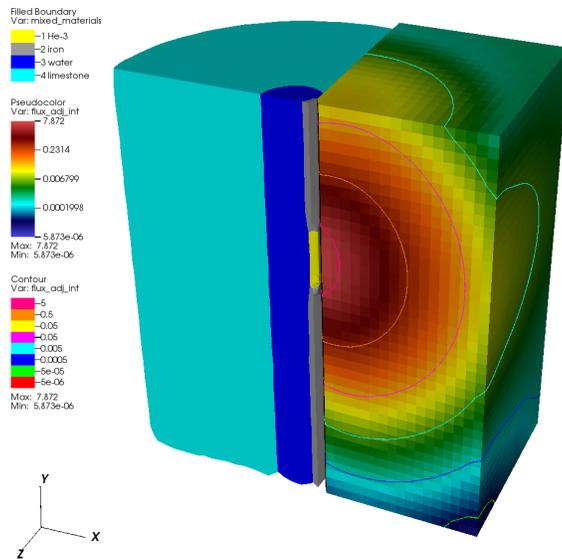


Fig. 3-7. Adjoint scalar flux in problem INP12.

It is strongly recommended to carefully examine the group-wise scalar flux solution(s) to determine if there are any significant nonphysical features (e.g., ray effects or negative fluxes). If necessary, the deterministic calculation(s) should be refined and the variance reduction parameters regenerated before proceeding to the Monte Carlo simulation.

3.8 VARIANCE REDUCTION PARAMETER GENERATION

ADVANTG provides computational sequences that generate variance reduction parameters using the CADIS and FW-CADIS methods. The parameters consist of space- and energy-dependent weight-window targets and a biased source that is consistent with the weight map. The total response, R , from Eq. (2-5) and the biased source probabilities, \hat{q} , from Eq. (2-9) are estimated by sampling the original (unbiased) source distribution and scoring the importance evaluated at the source particle's location and energy. The total response is the average score per source particle. The biased source probabilities are calculated as the average scores within the bins defined by the SI cards in the MCNP input file. Once the total response is known, weight-window targets are computed according to Eq. (2-8).

By default, ADVANTG automatically generates biased probabilities and outputs SB cards for all SDEF distributions that can be effectively biased with a space- and energy-dependent importance function. In most cases, probabilities will be generated only for spatial and energy distributions. However, if a variable (e.g., DIR) is depended upon by a spatial variable or the energy variable (ERG), biased probabilities will be generated for the independent variable as well.

In most problems, it is advantageous to partition the SDEF distribution bins in order to capture the spatial and spectral variations of the importance function in the biased source. Section 7 provides examples of how to generate effective source biasing parameters for several sample problems. Note that it is possible to recalculate the biased source probabilities after changing the SDEF distribution bins without re-running the discrete ordinates calculation(s). To do that, use the `-f` option when starting ADVANTG. (See Section 4 for a description of the ADVANTG command line options.)

When spatial source biasing is used with cell or cookie-cutter rejection, the mean weight of a source particle is in general no longer equal to one and the starting particle weight (WGT parameter of the SDEF card) must be corrected to preserve tally mean values. Because rejection is commonly used, an approach for estimating the corrected starting weight and its uncertainty has been developed and implemented in ADVANTG. The implementation is currently limited to the case in which rejection is applied using a single cell; rejection over multiple cells is not yet supported. For the single-cell case, ADVANTG calculates and outputs the corrected WGT value on a replacement SDEF card and its relative error in an adjacent comment card. Because the corrected starting weight directly multiplies all tally results, a warning message is printed if the relative uncertainty in the WGT estimate is greater than 1%. The uncertainty can be reduced by increasing the number of samples on the `mcnp_num_wgt_samples` card, which is described in Section 5.8.1. As with source biasing parameters, the WGT parameter can be recalculated without re-running the discrete ordinates calculation(s).

ADVANTG assumes that rejection is possible whenever the CEL or CCC keyword appears on the SDEF card. If it is truly impossible for a sampled source location to ever be rejected, then no correction to the starting particle weight is needed. However, ADVANTG will still estimate a corrected WGT parameter because it cannot generally know when rejection is impossible. In this case, ADVANTG will estimate a corrected WGT that is identical to the original starting weight with zero estimated relative error.

When cell or cookie-cutter rejection is possible, the corrected WGT parameter depends on the biased probabilities of the spatial variables as well as the biased probabilities of any variables on which the

spatial variables depend. For this reason, changing or removing the biased probabilities may invalidate the WGT parameter estimated by ADVANTG and lead to incorrectly normalized MCNP tally results.

Note that it is possible to disable the generation of biasing parameters for spatial variables, for the energy variable, or altogether. (See Section 5.8.1 for a description of the source biasing settings.) If rejection is possible but SB card generation is disabled for spatial variables, no weight correction is needed. In this case, ADVANTG will not estimate a corrected SDEF WGT parameter.

The WWINP file created by ADVANTG contains weight-window lower bounds calculated using Eq. (2-8) with R set to one. The value of R normalized to a unit source strength is set as the WNORM value on the corresponding WWP card(s). (WNORM is a lower-bound multiplier; it is the seventh parameter on the WWP card.) If the SDEF card in the MCNP input file specifies an initial particle weight other than one, then the WNORM value is multiplied by this modified starting weight.

Group-wise and energy-integrated weight-window targets can be displayed in VisIt, as shown in Fig. 3-8. The figure is analogous to Fig. 3-7, but it shows weight-window targets instead of importances. When the weight-window map is used in an MCNP simulation, particles moving toward the detector will undergo splitting, whereas particles moving away from the detector will be subjected to roulette.

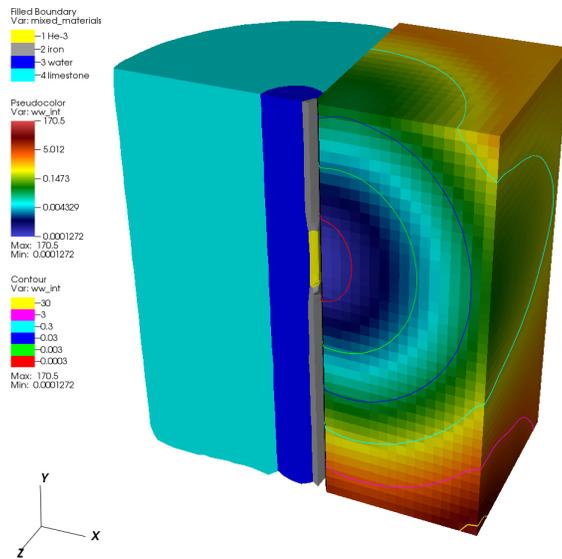


Fig. 3-8. Weight-window targets in problem INP12.

4. RUNNING ADVANTG

ADVANTG is executed from the command line using:

```
$ advantg [options] input_file
```

Command line options are listed in Table 4-1 below. The format and content of the input file is the subject of Section 5.

Table 4-1. Command line options

Option	Description
-h, --help	Print command line help and exit
--version	Print the code version and exit
--template [args]	Print a listing of the ADVANTG input keywords and associated descriptions. Arguments filter the keywords that are displayed.
-c, --clean	Ignore existing working subdirectories. The default is to reuse existing data unless input has changed.
-f, --force-resume	Force existing working data to be reused, even if it appears to be outdated
-g, --debug	Write extra information to disk for debugging purposes
-v, --verbose	Print all debug messages
-q, --quiet	Print only informational and warning messages
--very-quiet	Print only warning messages
--silent	Print messages only on failure
--log=LEVEL	Create a log file with the given verbosity level: DEBUG, STATUS, INFO, WARNING, ERROR, or CRITICAL

The current version of ADVANTG executes only in serial mode—that is, using a single thread/core. However, ADVANTG will invoke `mpiexec` to launch a parallel Denovo job when the product of `denovo_x_blocks` and `denovo_y_blocks` (the number of parallel domains) is greater than one. ADVANTG passes the number of domains as the argument to the `mpiexec -np` option. In addition, if a file named `machinefile` is found in the problem directory (the directory in which ADVANTG was launched), then ADVANTG will add a `-machinefile` option to the `mpiexec` command with the path of the file.

ADVANTG can drive parallel Denovo calculations on clusters with job submission systems. Consider a problem in which a Denovo will be executed across 36 domains. An example Portable Batch System (PBS) script is shown in Fig. 4-1 for a cluster with two hex-core processors per node. A PBS directive requests 3 nodes and 12 processes per node. Other directives set the amount of memory per process (1 GB), a 1-hour wall-clock limit, and a job name. The script sets the working directory and dumps the machinefile created by the batch system for this particular job before launching ADVANTG. When the job is started, ADVANTG begins running on a single core. When ADVANTG is ready to perform a

deterministic calculation, it invokes `mpiexec` to execute a parallel Denovo calculation using 36 cores across the three nodes that were reserved for the job.

```
#!/bin/bash
#PBS -N advantg
#PBS -l nodes=3:ppn=12
#PBS -l pmem=1gb
#PBS -l walltime=1:00:00
cd $HOME/calcs/case1/advantg
cat $PBS_NODEFILE > machinefile
advantg case1.adv
```

Fig. 4-1. Sample PBS script.

5. INPUT

ADVANTG reads user input from a free-format plaintext input file. The format of the input file is described in Section 5.1 below. The sections that follow describe the various input options that ADVANTG accepts. A typical input file will contain only a small fraction of the entries described in this section. Default values are defined for most options and only a few entries are required. For MCNP models, a minimal input consists of:

- The name of an MCNP input file,
- A list of one or more tally numbers,
- The name of a generator method (e.g., *cadis* or *fwcadis*),
- Lists of spatial mesh planes in the *x*, *y*, and *z* dimensions, and
- The name of a multigroup cross-section library.

ADVANTG extracts all model-related information (geometry, materials, sources, and tallies) automatically. As a result, input files tend to consist primarily of parameters for the deterministic calculation(s).

For new users, a recommended first step is to look at the example problems in Section 7. ADVANTG inputs are shown in Figs. 7-4, 7-13, and 7-23. This section can be referred to later for more detailed information. An index of frequently used keywords is given in Appendix B.

5.1 INPUT FILE FORMAT

ADVANTG input files are plaintext files organized into a series of entries or cards. Each entry begins with a case-insensitive keyword that must start within the first four columns of a line. The keyword must be followed by at least one value. Multiple values are separated by whitespace. Values can be one of the following types:

- Integer literals (e.g., 0, 123, -5)
- Floating point literals (e.g., 0.5, -1.2345e+6)
- Boolean literals (e.g., true, False, 0, 1, Y, n, T, f)
- Strings (e.g., gmres, "file name")

Strings containing spaces must be enclosed in double quotes (").

Entries can span multiple lines. Continuation lines are denoted by an indent of at least four columns. The length of input lines is not limited. Empty lines are ignored. Tab characters are expanded to eight spaces. With the exception of denoting a continuation line, multiple whitespace characters are treated the same as a single space.

Comments are denoted by the hash (#) character. All characters from the # character to the end of the line are ignored. Comments can start anywhere on a line and can be placed anywhere within an entry after the keyword.

5.2 DRIVER OPTIONS

The following high-level options determine which components are used to carry out the computational sequence:

model name (default: `mcnp`)

Type of transport problem model. One of:

<code>mcnp</code>	MCNP5 model
<code>sword</code>	SWORD model

method name (required)

Variance reduction method or type of computational sequence. One of:

<code>cadis</code>	CADIS method (single-tally variance reduction)
<code>fwcadis</code>	FW-CADIS method (multiple-tally variance reduction)
<code>dx</code>	Discretize the transport model and optionally execute deterministic calculation(s)

The `dx` option can be used to generate visualization output for the discretized model (see the `outputs` keyword below). This feature provides a means to quickly inspect the spatial mesh as well as the mapped materials, sources, and tallies, for example, as a preliminary step before proceeding to perform the transport calculations.

The `dx` option can also be used to execute a forward or adjoint deterministic calculation without generating variance reduction parameters. This feature is useful, for example, for performing scoping or feasibility studies or other calculations where continuous-energy Monte Carlo is not needed.

outputs name ... (default: `mcnp silo`)

Type(s) of output. Any of:

<code>mcnp</code>	MCNP weight-window file (WWINP) and biased source cards for the <code>cadis</code> or <code>fwcadis</code> methods
<code>silo</code>	Silo-format visualization files
<code>response</code>	Calculate, tabulate, and/or plot deterministic response estimates
<code>none</code>	No specialized output

The data within Silo-format files can be visualized using the VisIt open-source, 3-D, parallel visualization package. VisIt is an extremely powerful tool for exploring mesh-based data sets, and its use is highly recommended.

If the `response` option is selected, energy-dependent and energy-integrated responses will be calculated based on the deterministic transport solution. This option is most useful with the `dx` method.

5.3 MODEL OPTIONS

5.3.1 MCNP-Specific Options

The options and settings described in this section apply when `model` is `mcnp`. If a different model type is selected, these settings are ignored.

mcnp_input `filename` (required)

Filename of the MCNP5 input file. Enclose the filename in double quotes if it contains spaces.

mcnp_tallies `int ...`

MCNP tally number(s) for which to generate variance reduction parameters and/or to use in forming adjoint sources. This keyword is required if a deterministic adjoint calculation will be performed.

mcnp_material_names `int name ...`

Names of MCNP materials for output purposes, listed as pairs of MCNP `m` card numbers and name strings. If the `sil` output option is selected, the material names will be written to the Silo file and will appear in VisIt. By default, material 0 is named `void`.

Example:

```
mcnp_material_names  1  Air
                    10 He-3
                    11 "Stainless steel"
```

The following parameters control the number of samples drawn from the MCNP SDEF source when mapping it onto the spatial mesh. Note that these parameters do not affect the calculation of biased source distributions. (Instead, see the keywords described in Section 5.8.1).

mcnp_min_source_samples `int >= 0` (default: 1e6)
mcnp_max_source_samples " (default: 1e8)

Minimum and maximum number of times to sample the SDEF source.

mcnp_target_source_density `int >= 0` (default: 100)

Target minimum average number of source samples per mesh voxel containing source. If this target is not reached after `mcnp_min_source_samples` have been drawn, up to `mcnp_max_source_samples` will be generated in an attempt to reach this target.

A warning message will be printed if the target is not reached.

mcnp_max_point_sources `int >= 0` (default: 20)

Maximum number of point sources. If the SDEF source consists of more than this number of point sources, it will be mapped as a volume source for the deterministic calculation.

Each point source increases the run time and memory consumption of the Denovo calculation. This option exists to avoid executing a deterministic calculation that may have an unexpectedly large memory requirement or a long run time due to the number of first-collision sources.

mcnp_force_point_source `bool` (default: False)

Treat the source as a single point source in the deterministic calculation?

It is generally recommended to treat a small volume source (relative to the size of the problem domain) as a point source in the discrete ordinates calculation in order to activate a first-collision source treatment. This option provides a means to do this without modifying the original SDEF cards.

The following parameters control the number and direction of rays traced through the MCNP geometry to map materials and tally regions onto the spatial mesh.

mcnp_min_rays_per_face `int > 0` (default: 10)

Minimum number of rays to trace through each voxel face in each trace direction.

mcnp_ray_directions `axis ...` (default: x y z)

Directions in which to trace rays through the MCNP geometry model. Rays are traced parallel to the spatial axes and in the positive direction. Any combination of x, y, and/or z can be selected.

mcnp_num_rays `int > 0` (default: 1)

Nominal total number of rays.

The nominal number of rays to be traced through each voxel face is calculated in proportion to the area of the face. The per-face minimum is then applied so that the actual number of rays is the greater of the nominal number and the `mcnp_min_rays_per_face` setting (10 rays, by default).

By default, the `mcnp_min_rays_per_face` setting determines the total number of rays that are actually sampled. The `mcnp_num_rays` setting is provided because it can be useful in problems where the spatial mesh voxels vary greatly in size.

mcnp_lost_rays `int` (default: 10)

Maximum number of lost rays allowed while mapping material and tally regions onto the spatial mesh. An error message is printed if the limit is reached.

mcnp_mix_tolerance `real >= 0.0` (default: 0.01)

Tolerance for combining mixed materials. Materials with component volume fractions that differ by less than or equal to this amount will be combined to limit the total number of mixed materials generated. Higher numbers reduce the amount of memory needed to store material mixtures. Lower numbers provide more accurate per-voxel discretized material compositions.

For example, if voxels A, B, and C contain 70, 90, and 91% steel by volume, respectively, and 30, 10, and 9% air, then by default, voxels B and C will contain the same mixed material. Voxel A will contain a different mixture.

The following parameters control the unfolding options that can be used if the MCNP5 model has reflecting boundaries.

mcnp_unfolding_origin `real(3)`

Specifies the x , y , and z coordinates of a reference point inside a reflected MCNP geometry that is to be unfolded for the Denovo deterministic calculations. Avoid placing the reference point near external surfaces.

Note that the rays used to perform the unfolding are terminated when they reach a cell of zero importance. For certain geometries, it may be necessary to extend the model with one or more voided cells that have nonzero importance in order for the unfolding to work properly. In all cases, careful inspection of the unfolded geometry in VisIt is highly recommended.

mcnp_unfolding_safe `bool` (default: False)

If True, activates a “safe” ray tracing mode for the unfolding process. This safe mode is slower than the default mode, but the probability of generating lost rays is lower. Use the safe mode only if there are issues with lost rays when using the default mode.

5.3.2 SWORD-Specific Options

The options and settings described in this section apply when `model` is set to `sword`. If a different model type is selected, these settings are ignored.

sword_input `filename` (required)

Filename of the `.sword` file. Enclose the filename in double quotes if it contains spaces.

sword_mix_tolerance `real` (default: 0.01)

Tolerance for combining mixed materials (analogous to `mcnp_mix_tolerance`). Materials with component volume fractions that differ by less than or equal to this amount will be combined to limit the total number of mixed materials generated. Higher numbers reduce the amount of memory needed to store material mixtures. Lower numbers provide more accurate per-cell discretized material compositions.

sword_small_sources `bool` (default: True)

If True, single-voxel sources are approximated as a point source of equivalent strength to activate the first-collision source treatment in Denovo. Generally, this substitution yields more accurate deterministic results.

sword_sampling `name` (default: subcell)

Sampling algorithm to be used for discretizing the SWORD geometry. One of:

subcell	Specify the number of samples per cell
res	Specify the sampling resolution

sword_subcell `int` (default: 2)

sword_subcell_x	“
sword_subcell_y	“
sword_subcell_z	“

When `sword_sampling` is `subcell`, the `sword_subcell` keyword sets the number of point samples per interval in each spatial dimension. The dimension-specific keywords can be used to override this setting for a particular dimension.

For example, when `sword_subcell` is 1, material compositions are based on the material found at the center of each voxel. When the value is 2, eight samples per voxel are used to generate a mixed-material description of the contents of the voxel.

sword_resolution `float`

When `sword_sampling` is `res`, sets the length, in cm, of the desired point sampling interval for each spatial dimension. For example, if `sword_resolution` is set to 2.0, then in a voxel with dimensions $10 \times 6 \times 4$ cm, 5 point samples will be taken along the x dimension, 3 along y , and 2 along z , for a total of 30 samples within the voxel.

5.4 METHOD OPTIONS

No CADIS-specific input parameters are currently defined.

5.4.1 FW-CADIS-Specific Options

fwcadis_spatial_treatment `name` (default: `pathlength`)

Spatial treatment used when constructing the FW-CADIS adjoint source. One of:

`pathlength` Use path-length weighting
`global` Use global weighting

The `pathlength` treatment is recommended for problems with multiple cell, surface, and/or point detector tallies. The `global` treatment is recommended for mesh tallies. See Sections 2.3.1 and 2.3.2 for more details.

fwcadis_response_weighting `bool` (default: `True`)

If `True`, construct the adjoint source to accelerate an energy-integrated response (e.g., dose or total flux). If `False`, generate parameters with the objective of obtaining relatively uniform uncertainties across all energy groups. See Sections 2.3.3 and 2.3.4 for more details.

Note that turning off response weighting generally increases the time per history in the Monte Carlo simulation, especially if the group-wise flux in the region of interest varies over many orders of magnitude.

fwcadis_min_response `real >= 0.` (default: `0`)
fwcadis_max_response `"` (default: `infinite`)

Lower and upper energy-integrated response bounds for including voxels when constructing the FW-CADIS adjoint source with the `global` spatial treatment. By default, no bounds are used.

This feature can be useful, for example, when estimating dose rate maps. In practical applications, there is generally little interest in dose rates that fall below a certain level. By excluding the adjoint source from the region outside of a particular dose rate contour, a more efficient Monte Carlo simulation is obtained.

Note that the locations of the upper and lower bounds are calculated based on the deterministic solution, which might not be highly accurate. It is therefore recommended to set the lower/upper bounds somewhat lower/higher than the actual contour of interest (e.g., by one or two orders of magnitude). To ensure that the thresholds are properly chosen, the responses calculated using the deterministic and MCNP models should be compared. If the difference between these responses is larger than the margin applied to the bound values, those values should be adjusted and the ADVANTG calculation should be re-run.

5.4.2 DX-Specific Options

The `dx` option provides a means to discretize the model and optionally execute deterministic calculations without generating variance reduction parameters. The options described below control which deterministic calculations are performed, if any.

`dx_adjoint` `bool` (default: False)
`dx_forward` “ “

If `True`, executes a deterministic calculation in the specified transport mode. By default, no deterministic calculations are executed; the model is discretized, and the quality of the result can be visualized using the appropriate output options (see Section 5.8.2).

5.5 SPATIAL MESH

The following settings are used to construct a 3-D rectangular spatial mesh as the product of 1-D meshes in the x , y , and z dimensions. By default, the mesh of the weight-window map is the same as that used in the deterministic calculations.

`mesh_refinement` `name` (default: `mcnp`)

Technique for refining the mesh. One of:

`mcnp` Use the `mesh_x_ints`, `mesh_y_ints`, and `mesh_z_ints` keywords
`uniform` Use the `mesh_max_width` and `mesh_min_width` keywords

The `mcnp` option uniformly subdivides each mesh interval. The `uniform` option adds and/or removes voxel edges in an attempt to generate nearly uniform widths across each dimension.

`mesh_x` `real ...` (required)
`mesh_y` “ “
`mesh_z` “ “

Coordinates of mesh voxel edges in the x , y , and z dimensions, in cm. At least two edges per dimension are required. The edge coordinates must be listed in increasing order.

The mesh can be refined using one of two approaches described with the `mesh_refinement` keyword. By default, the mesh intervals are not subdivided further and the `mesh_x`, `mesh_y`, and `mesh_z` cards define the final mesh.

`mesh_x_ints` `int > 0 ...` (default: 1 subdivision per mesh interval)
`mesh_y_ints` “ “
`mesh_z_ints` “ “

The number of uniform subdivisions in each mesh interval when `mesh_refinement` is `mcnp`. The number is ignored if `mesh_refinement` is not `mcnp`. By default, the original mesh intervals are not subdivided further. The number of entries must be one less than for `mesh_x`, `mesh_y`, or `mesh_z`.

Example:

```
mesh_x      0  10  20
mesh_x_ints      2  4
```

These cards define a spatial grid in the *x*-dimension that has voxel edges at the coordinates: 0, 5, 10, 12.5, 15, 17.5, and 20 cm.

```
mesh_max_width  real > 0.
mesh_x_max_width  "
mesh_y_max_width  "
mesh_z_max_width  "
```

Maximum distance between voxel edges, in cm, when *mesh_refinement* is uniform. This is required if *mesh_refinement* is uniform and ignored otherwise. The *mesh_max_width* keyword sets the default maximum width for all dimensions. The dimension-specific keywords can be used to override this setting for a particular dimension.

```
mesh_min_width  real >= 0. (default: 0.0)
mesh_x_min_width  "
mesh_y_min_width  "
mesh_z_min_width  "
```

Minimum distance in cm between voxel edges when *mesh_refinement* is uniform. This is ignored if *mesh_refinement* is not uniform. The *mesh_min_width* keyword sets the default minimum width for all dimensions. The dimension-specific keywords can be used to override this setting for a particular dimension.

By default, the minimum voxel width is zero, and none of the grid points specified on the *mesh_x*, *mesh_y*, or *mesh_z* cards will be removed.

Example:

```
mesh_x      -10 -5.5  0  0.25  5.5  10
mesh_min_width  0.75
mesh_max_width  2
```

These input cards will generate an *x*-dimension spatial grid that has voxel edges at: -10, -8.5, -7, -5.5, -3.67, -1.83, 0, 1.83, 3.67, 5.5, 7.0, 8.5, and 10 cm.

Note that the original edge at 0.25 cm has been removed because *mesh_min_width* is 0.75 cm. This can be avoided by setting the minimum width to 0.25 cm or smaller.

5.6 MULTIGROUP LIBRARY OPTIONS

ADVANTG reads multigroup cross section data from ANISN-format libraries. The options described in this section can be used to select a cross section library, set the upscatter treatment, and customize mappings of ZAIDs to cross section tables.

anisn_library `name` (required)

Name of the ANISN-format cross section library (see Section 3.2 for a list of library names). The search path defined at configuration time is searched to locate the library and associated metadata files.

anisn_upscatter `bool` (default: False)

If False, then treat upscatter as self-scatter and avoid costly upscatter iterations in the Denovo calculation(s). If the accuracy of thermal neutron fluxes is important, then consider setting `anisn_upscatter` to True. This option has no impact on photon-only problems.

anisn_zaid_map `int > 0 ...`

Additional ZAID and ANISN table ID pairs to use when mapping the MCNP material compositions to multigroup materials for the deterministic calculation. An arbitrary number of pairs can be given. Entries on this card add to or override the mappings defined in the `.zaid` file associated with the selected ANISN library.

Example:

```
anisn_library      27n19g
anisn_zaid_map    1001  1
                  1002  7
```

In the 27n19g library, nuclides 1001 (H-1) and 1002 (H-2) are mapped to cross section tables for bound hydrogen in H₂O (table 2395) and D₂O (table 2383), respectively. In this example, these defaults are overridden to map 1001 and 1002 to cross section for unbound H-1 (table 1) and H-2 (table 7). Note that this example is intended for illustrative purposes only. Only in rare cases is it desirable to use unbound hydrogen cross sections.

5.7 SOLVER OPTIONS

The Denovo discrete ordinates solver provides a variety of options and settings. For generating variance reduction parameters, typically only a fraction of these settings are used or modified. The most frequently used parameters are:

- `denovo_quadrature` and associated options, which select and configure the S_N angular approximation,
- `denovo_pn_order`, which determines the degree of scattering anisotropy modeled, and
- `denovo_x_blocks` and `denovo_y_blocks`, which determine the parallel spatial decomposition (if Denovo was built with an MPI library).

The amount of memory used by the Denovo solver state (i.e., by the discrete ordinates solution in memory) is

$$\text{state size} = N_v(N_g + N_K)(L + 1)^2 N_u \cdot 8 \text{ bytes}, \quad (5-1)$$

where N_v is the number of mesh voxels, N_g is the number of solved energy groups, N_K is the number of Krylov vectors (default is 20), and L is the scattering expansion order. N_u is the number of unknowns per voxel and is determined by the spatial discretization scheme (see below). The factor of eight is because the solver state is stored in double precision. First-collision and FW-CADIS adjoint sources may consume significant amounts of additional memory. In addition, a small overhead (for communication buffers, local-to-global maps, etc.) is added for each process when Denovo is executed in parallel. Thus, Eq. (5-1) represents a lower limit, not an upper bound. For cluster jobs, the number of nodes and/or the number of cores per node should be selected so that the memory usage per node does not exceed the available memory (with an allowance for the operating system and any other running jobs).

The following options and settings control the spatial discretization and parallel spatial decomposition:

denovo_discretization name (default: **sc**)

Spatial discretization. One of:		N_u
ld	Linear discontinuous	4
sc	Step characteristics	1
tld	Trilinear discontinuous	8
twd	Theta-weighted diamond difference	1
wdd	Diamond difference	1
wdd_ff	Diamond difference with negative flux fixup	1

The step-characteristics discretization scheme is robust with regard to mesh size and voxel aspect ratios and always produces positive fluxes given positive sources. It has first-order accuracy (i.e., the discretization error decreases linearly with mesh size in the asymptotic limit) and requires storage of only one unknown per voxel in memory. Because of its robustness, positivity, and low memory requirements, it is the default scheme in ADVANTG. Note that the step-characteristics method will tend to overestimate the fluxes deep within an attenuating material.

The linear and trilinear discontinuous (LD and TLD) schemes have second-order accuracy, but do not ensure positivity. The LD and TLD methods require storage of four and eight unknowns per voxel, respectively, and they also incur a similar increase in run time. These discretization schemes tend to give very accurate results on well-refined meshes but can break down on coarser grids. The LD scheme tends to be rather sensitive to the aspect ratio of the mesh voxels. The TLD scheme is more robust than LD, but it incurs a significant cost relative to step characteristics.

The diamond difference and theta-weighted diamond techniques have a long history. The weighting helps to dampen but does not entirely eliminate the oscillations exhibited by diamond difference solutions in 3-D problems. It also reduces the order of accuracy to somewhere between first and second order. Because the step characteristics scheme provides significantly better solutions in most problems, the use of the diamond difference technique is generally discouraged.

```
denovo_x_blocks int > 0 (default: 1)
denovo_y_blocks  "          "
denovo_z_blocks  "          "
```

The number of spatial domain partitions in the x and y dimensions and the number of pipelining blocks in the z dimension.

The Koch-Baker-Alcouffe 3-D parallel sweep algorithm is implemented in Denovo on a domain that is decomposed into x - y blocks. Each parallel core is assigned a single x - y block. So for example, with `denovo_x_blocks` and `denovo_y_blocks` both set to 2, a parallel calculation across four cores is performed. The number of blocks in each dimension should be chosen in proportion to the number of mesh intervals.

The x - y blocks can be divided into multiple blocks in the z dimension. The z dimension blocks do not affect the domain decomposition (or the number of cores used), but they determine the frequency at which fluxes are communicated between neighboring domains. This setting can impact performance in problems that have a large number of intervals in the z dimension.

By default, a single-core calculation is performed with no domain decomposition. Note that Denovo must be built with an MPI library in order to execute parallel calculations.

The following options control the partitioning of the energy groups across processors:

```
denovo_energy_sets int > 0 (default: 1)
```

The number of parallel partitions in energy.

The parallel performance of the solver scales well with the number of spatial domain blocks. At some point, however, the domain blocks are small enough that communication overhead reduces efficiency. Energy partitioning provides a means to maintain solver efficiency across a larger number of cores. The number of executing cores is the product of the number of x - y domain blocks and the number of energy sets. This feature is generally needed only in massively parallel S_N calculations.

```
denovo_partition_upscatter bool (default: False)
```

If `True`, partition just the upscatter groups. This option is only valid when more than one energy set is used and when `anisn_upscatter` is `True` (see Section 5.6).

The following options and settings control the S_N angular quadrature:

```
denovo_quadrature name (default: qr)
```

Type of quadrature set. One of:

<code>glproduct</code>	Gauss-Legendre product
<code>ldfe</code>	Linear-discontinuous finite element (triangular)
<code>levelsym</code>	Level symmetric (triangular)
<code>qr</code>	Quadruple range (product or triangular)
<code>userdefined</code>	User-defined set

Denovo provides several types of angular quadratures which fall into two basic classes: triangular and product. Triangular quadrature sets are characterized by an integer order (see the `denovo_quad_order` or `denovo_ldfe_order` keywords). Product quadratures are determined by the number of azimuthal and polar angles per octant (see the `denovo_quad_num_azi` and `denovo_quad_num_polar` keywords). Triangular quadrature sets are generally rotationally symmetric, whereas product quadrature sets are generally not. However, product quadratures provide the flexibility to vary the number of azimuthal and polar angles independently, which can be more efficient in some types of transport problems.

Note that the run time of the S_N calculations scales nearly linearly with the number of quadrature angles (all else being equal). For triangular quadratures, the number of angles does not increase linearly with the order (see below). Memory usage is independent from the number of angles, because only the $(L + 1)^2$ angular flux moments are retained in memory.

Gauss-Legendre (GL) product quadratures are formed as the Cartesian product of a set of uniformly distributed azimuthal angles and a 1-D Gauss-Legendre quadrature in the polar angle.

Linear-discontinuous finite element (LDFE) quadratures (Jarrell 2010) are based on an approach that approximates the angular flux as $\psi \approx c_1 + c_2\mu + c_3\eta + c_4\xi$, where μ , η , and ξ are the direction cosines with respect to the x , y , and z coordinate axes. The quadrature is determined by requiring that the integration of the basis functions equals the surface area of the unit sphere. The LDFE sets have positive weights and are rotationally symmetric about all three axes. There are $4^{(N+1)}$ angles per octant, where N is the order. Unlike level-symmetric quadratures, the LDFE order can be even or odd. The maximum LDFE order in Denovo is seven.

Level-symmetric quadratures have a long history with the S_N method. They are rotationally-symmetric quadratures that have positive weights up to S_{20} . In 3-D, there are $N(N + 2)/8$ angles per octant, where N is the order. The maximum order supported by Denovo is 24. The order must be an even number.

Quadruple range (QR) product quadratures (Abu-Shumays 2001) exactly integrate maximal-order products of sines and cosines of the polar and azimuthal angles. The maximum numbers of azimuthal and polar angles per octant for QR quadratures in Denovo are 37 and 16, respectively. The QR product quadratures perform well across a broad range of transport problems and tend to exhibit far less ray effects than level-symmetric quadratures. For these reasons, the QR product set with four azimuthal and four polar angles per octant is the default quadrature used by ADVANTG. This set is often sufficient for generating variance reduction parameters for neutron-only problems. For photon-only problems or Denovo-only calculations, a more refined quadrature set is generally recommended.

Denovo also implements the QR quadrature as a triangular type quadrature that contains the same number of angles as a level-symmetric quadrature set of the same order. However, the triangular QR quadrature is not rotationally symmetric. The maximum order is 32.

For nonsymmetric quadratures (GL and QR), the default polar axis is the z -axis. A different polar axis can be selected using the `denovo_quad_polar_axis` keyword.

Denovo also provides the facility to read in user-provided quadratures. See the `denovo_quad_file` keyword below.

denovo_quad_order `even int > 1` (default: 10)
denovo_ldfe_order `int > 0` (default: 1)

Triangular quadrature orders. The `denovo_quad_order` keyword sets the order for level-symmetric and triangular QR sets. The maximum orders implemented in Denovo for the level-symmetric, QR, and LDFE quadratures are 24, 32, and 7, respectively. Note that for orders greater than 20, the level-symmetric quadratures will contain some negative ordinate weights.

denovo_quad_num_azi `int > 0` (default: 4)
denovo_quad_num_polar `"` `"`

Number of azimuthal and polar directions per octant for product quadrature sets. The maximum numbers of azimuthal and polar angles per octant for QR quadratures are 37 and 16, respectively. There are no such limits for the GL product quadrature.

denovo_quad_num_azi_vec `int > 0 ...`

For product quadrature sets, the number of azimuthal angles per polar angle per octant, ordered from the polar axis toward the equator. If specified, this option takes precedence over `denovo_quad_num_azi`.

Example:

```
denovo_quadrature      qr
denovo_quad_polar_axis z
denovo_quad_num_polar  4
denovo_quad_num_azi_vec 3 4 5 6
```

This example specifies a QR product quadrature set with four polar angles per octant, a variable number of azimuthal angles, and a total of 144 ($= 18 \times 8$) directions. The polar levels with maximum and minimum cosines with respect to the positive z -axis contain three and six azimuthal angles per octant, respectively.

denovo_quad_polar_axis `axis ...` (default: z)

Polar axis for nonsymmetric quadrature sets. Must be one of: x, y, or z.

denovo_quad_file `filename`

Filename of a user-provided quadrature file. This is ignored unless `denovo_quadrature` is `userdefined`.

The quadrature file is a free-format plaintext file. The direction cosines and weight of each quadrature direction are listed on a single line as: $\mu_n \eta_n \xi_n w_n$. The direction cosines must be normalized so that $\mu^2 + \eta^2 + \xi^2$ is within 1 ± 10^{-6} . The weights must be strictly greater than zero. Lines with a hash character (#) in the first column are ignored.

The following options and settings control the treatment of scattering. If `anisn_upscatter` is `False` (the default), no upscatter iterations are performed. Upscatter iterations are performed if `anisn_upscatter` is `True` and the multigroup library contains upscatter data.

denovo_pn_order `int >= 0` (default: 3)

Order of the Legendre scattering-angle expansion. Denovo supports scattering expansions up to 11th order. Most multigroup cross section libraries have scattering data up to 5th or 7th order.

The memory consumed by Denovo scales as $(L + 1)^2$, where L is the scattering order. Thus, a P_3 calculation consumes four times more memory than a P_1 calculation.

Generally, a relatively high expansion order (P_3 and above) is recommended for photon and coupled neutron-photon calculations. A lower order can often lead to a negative scattering source (caused by truncation error); this can produce negative fluxes, even with the step characteristic differencing scheme. Negative fluxes are still possible with a higher order expansion; however, the magnitudes of any negative scattering sources are generally much lower. It is possible to enforce a positive scattering source (see `denovo_transport_correction`) at the expense of accuracy.

denovo_transport_correction `name` (default: diagonal)

Transport correction. One of:

<code>cesaro</code>	Cesaro positive-preserving
<code>diagonal</code>	$\hat{\sigma}_g = \sigma_g - \sigma_{sL+1}^{gg}$ $\hat{\sigma}_{sl}^{gg} = \sigma_{sl}^{gg} - \sigma_{sL+1}^{gg}, l = 0, \dots, L$
<code>none</code>	No transport correction

The diagonal transport correction should provide accurate results for most problems. The Cesaro correction ensures a positive scattering source, but at the expense of accuracy. It can be used if `denovo_pn_order` is at least 2.

Denovo contains two embedded first-collision source capabilities: an analytic point-kernel treatment and a Monte Carlo implementation for distributed sources. The Monte Carlo implementation traces rays from randomly sampled starting points in random directions. It can only be used with sources that have a single energy spectrum.

Point sources on the MCNP SDEF card are handled with the analytic first-collision source. The Monte Carlo first-collision source treatment must be explicitly activated for distributed sources using the following options and settings:

denovo_mc_first_collision `bool` (default: False)

If `True`, enables the Monte Carlo first collision source for distributed sources. By default, no first-collision source treatment is applied to distributed sources.

denovo_mc_num_particles `int > 0` (default: 10000)

Number of particles to simulate with the Monte Carlo first collision treatment.

The following options and settings control the within-group and upscatter solvers:

denovo_solver `name` (default: gmres)

Within-group solver. One of:

gmres	Restarted GMRES solver
si	Source (Richardson) iteration

The GMRES solver generally converges more quickly and is able to converge to tighter tolerances than source iteration, especially for problems with thick scattering media.

denovo_multigroup_solver `name` (default: gauss_seidel)

Upscatter solver. One of:

gauss_seidel	Gauss-Seidel solver
gmres	Restarted GMRES solver

This option is ignored if `anisn_upscatter` is `False`. The GMRES upscatter solver will generally converge more rapidly than Gauss-Seidel iterations. However, the GMRES option is not currently compatible with either first-collision source implementation.

denovo_preconditioner `name` (default: none)

Preconditioner for the within-group solver. One of:

dsa	Diffusion synthetic acceleration
none	No preconditioner

denovo_two_grid `bool` (default: False)

If `True` and if `anisn_upscatter` is `True`, enables two-grid upscatter acceleration.

denovo_krylov_space `int > 0` (default: 20)

Maximum number of Krylov vectors to store for the GMRES within-group solver.

Reducing the number of vectors reduces the memory consumed by Denovo, particularly in problems with a relatively small number of solved energy groups (e.g., as in photon-only calculations). Setting the number of vectors too low can increase the number of within-group iterations needed to reach the convergence criterion or prevent convergence altogether. Increasing the number of Krylov vectors beyond the default value typically has no benefit.

denovo_max_iterations `int > 0` (default: 100)

Maximum number of within-group iterations.

The GMRES solver will generally converge well before the default upper limit is reached, even in problems with thick scattering materials. Nonetheless, the number of actual iterations per group should be studied to ensure that all within-group solves were converged.

denovo_tolerance `real > 0.` (default: 0.001)

Convergence criterion for the within-group iterations.

The convergence measure used by Denovo is the L_2 norm of the residual vector for the GMRES solver and the L_∞ norm for the source iteration solver. These measures are not equivalent to the pointwise relative flux difference that is used in some other transport codes. The criterion of 10^{-3} has generally been found to be sufficient for the purpose of generating variance reduction parameters. For Denovo-only calculations, it is recommended to set this limit to 10^{-5} or lower.

denovo_upscatter_tolerance `real > 0.` (default: 0.01)

If `anisn_upscatter` is True, the convergence tolerance for the upscatter source.

denovo_upscatter_inner_iterations `int > 0` (default: 10)

If `anisn_upscatter` is True, the maximum number of iterations for each within-group solve in the upscatter iteration.

denovo_upscatter_inner_tolerance `real > 0.` (default: 0.01)

If `anisn_upscatter` is True, the tolerance for each within-group solve in the upscattering iteration.

denovo_first_group `int >= 0` (default: 0)

denovo_last_group `int >= 0`

The first and last energy groups to be solved. Note that in Denovo and ADVANTG, energy groups are indexed from zero rather than one. See Appendix A for lists of the group boundaries of the cross section libraries distributed with ADVANTG.

By default, the last group is the lowest energy neutron group in neutron-only problems and the lowest energy photon group in photon-only and coupled neutron-photon problems.

The following option(s) control the output of Denovo:

denovo_verbose `bool` (default: True)

If True, extra solver output (debug information, convergence information, and input configuration) is written to the `stdout` file when Denovo is executed.

5.8 OUTPUT OPTIONS

5.8.1 MCNP-Specific Options

mcnp_input_template `filename`

For CADIS or FW-CADIS calculations, the filename of an alternate MCNP5 input file to use as a basis for generating a new MCNP input with biased source (SB) and weight-window control parameter (WWP) cards. The template file is not modified. By default, `mcnp_input` is used. The new input file, named `inp`, is written to the `output/` directory.

This feature is useful when it is necessary to modify the MCNP input just for the purpose of running ADVANTG.

mcnp_mxspln `int >= 2` (default: 100)

Maximum number of weight-window splits per event. This parameter is written as the `MXSPLN` entry of the `WWP` card(s) in the generated MCNP input file.

mcnp_ww_ratio `real >= 2.0` (default: 5.0)

Ratio of the upper to lower weight-window bounds. This parameter is written as the `WUPN` entry of the `WWP` card(s) in the generated MCNP input file.

mcnp_sb_type `name` (default: `space_energy`)

Type of biased source probabilities to generate. One of:

<code>space_energy</code>	Generate SB cards for both spatial and energy distributions
<code>space</code>	Generate SB cards for spatial distributions only
<code>energy</code>	Generate SB cards for energy distributions only
<code>none</code>	Do not generate biased source probabilities

This option can be used, for example, to disable the generation of biased source cards for spatial distributions in problems with cell or cookie-cutter rejection in order to avoid the need to use a corrected starting particle weight.

mcnp_min_sb_samples `int >= 0` (default: 1e6)
mcnp_max_sb_samples `int >= 1` (default: 1e8)

Minimum and maximum number of times to sample the SDEF source when estimating biased source probabilities.

mcnp_target_sb_density `int >= 0` (default: 1e4)

Target minimum average number of samples per SDEF distribution bin when estimating biased source probabilities.

mcnp_user_sb_sampling `bool` (default: False)

If true, use the SB cards in the original MCNP input file when sampling the SDEF source to estimate biased source probabilities.

By default, any input SB cards are ignored, and the bins of the distributions to be biased are sampled uniformly. In certain cases, for example with rejection or with heavily unbalanced distribution trees, this approach may produce an insufficient number of samples in some bins. This setting can be used to override the uniform bin sampling and provide arbitrary probabilities for any distribution or set of distributions.

mcnp_num_wgt_samples `int >= 2` (default: 1e7)

Number of times to sample the SDEF source when estimating a corrected SDEF WGT parameter. This option is needed only when spatial source biasing is used with cell or cookie-cutter rejection.

mcnp_ww_collapse_factor `real >= 1` (default: 1)

Factor by which to reduce the size of the output weight-window map. This feature is useful in problems that would otherwise generate a weight-window map larger than MCNP can handle.

The collapsing algorithm post-processes the weight-window map to remove mesh planes and/or energy boundaries in phase-space regions that contribute the least to the response of interest, as determined by the product of forward and adjoint scalar fluxes. For CADIS problems, experience indicates that tally FOMs are generally insensitive to reductions in weight-window map size using this approach. However, this is not the case for FW-CADIS problems with mesh tallies that span a significant portion of the problem domain (e.g., as in the problem described in Section 7.3). For this reason, use of this feature is not recommended in that type of problem.

5.8.2 Silo-Specific Options

silos_response_ids `bool` (default: True)
silos_source_ids “ “

If True, writes maps of individual response or source identifiers to the Silo output file.

silos_source_strength `bool` (default: True)

If True, writes the distribution of the volume-averaged, energy-integrated source strengths to the Silo output file.

silos_ww `bool` (default: False)

If True, writes an expression for the weight-window targets to the Silo output file. The expression has a negligible effect on the size of the Silo file. This option is only valid when an adjoint solution has been calculated.

silos_edit_reactions `int name ...`

Specifies one or more edit fields to be calculated and written to the Silo output file, listed as pairs of edit reaction numbers and name strings. Edit reaction cross sections (or dose functions, fission spectra, etc.) are located in the ANISN cross section table in positions 1 to (IHT - 1), where IHT is the location of the total cross section. The number of edits available and the contents of the edits are library specific.

Note that Silo field names can contain only alphanumeric characters and underscores (_).

Example:

```
silos_edit_reactions 1 nu_sigma_f
```

6. OUTPUT

The primary output from ADVANTG is a WWINP file containing space- and energy-dependent weight-window lower bounds and the user's MCNP input file extended with biased source distributions and weight-window control parameters. Both files are written in a format compatible with unmodified versions of MCNP. ADVANTG also outputs status, warning, and error messages (if any) to the terminal window. Status messages are intended to indicate the progress of the computational sequence. Warning and error messages indicate potential or actual problems in inputs or computational results.

ADVANTG has the capability to write materials, scalar flux fields, sources, and responses in Silo format. Silo is an open-source library that provides an interface for reading and writing mesh-based data fields to a binary file, or to multiple files in parallel. The data in Silo files can be visualized using the VisIt visualization software. This functionality provides a means for the user to review the discretized model and flux estimates generated by Denovo. The user should carefully study this information to determine if obvious nonphysical features are present in important parts of the solution. If so, then input parameters must be modified to produce a more accurate deterministic calculation.

When ADVANTG is executed, it creates several directories below the current working directory. A list of the directories that can be created, depending on the calculation sequence, is shown in Table 6-1. In these directories, ADVANTG stores nearly all of the data that it generates, as well as input and output files for the codes that it executes. Much of this information, with the exception of large data arrays, is in a human-readable format that can be reviewed and inspected in case any problems arise. By default, ADVANTG will reuse as much of this data as possible if it is executed again. For example, if a tally is added to an FW-CADIS calculation, then the source mapping and forward Denovo calculation can be reused and will not be re-executed.

Table 6-1. Problem sub-directories

Directory	Description
<code>adj_solution/</code>	Denovo working directory for adjoint discrete ordinates calculations with the <code>cadis</code> and <code>dx</code> methods.
<code>fwd_solution/</code>	Denovo working directory for forward discrete ordinates calculations with the <code>fwcadis</code> and <code>dx</code> methods.
<code>fwcadis_adj_solution/</code>	Denovo working directory for adjoint discrete ordinates calculations with the <code>fwcadis</code> method.
<code>model/</code>	Working directory for model discretization tasks. (For MCNP models, MCNP is executed in this directory to create the <code>runtpc</code> file from which ADVANTG extracts model information.)
<code>output/</code>	Output directory containing the <code>WWINP</code> file and modified MCNP input file if variance reduction parameters were calculated, the VisIt-viewable <code>fields.silo</code> file, and the <code>status.log</code> file with all status, warning, and error messages.

7. EXAMPLES

This section presents three example problems that demonstrate the use of ADVANTG to generate variance reduction parameters for continuous-energy MCNP5 simulations. The first example uses the CADIS method to accelerate a point detector tally. The second and third examples use the FW-CADIS method to obtain relatively uniform statistical uncertainties across the energy bins of a pulse-height tally and the voxels of a mesh tally, respectively. MCNP and ADVANTG input files for all three example problems can be found in the `examples/` directory of the ADVANTG distribution.

7.1 UEKI SHIELDING EXPERIMENTS

7.1.1 Background

Ueki et al. (1992) describe a series of experimental measurements that were taken to investigate the neutron shielding properties of several materials. A schematic of the experiment is shown in Fig. 7-1 (reproduced from Ueki et al., Fig. 2). In the figure, all dimensions are listed in cm. A ^{252}Cf neutron source with a strength of 4.05×10^7 n/s was placed at the center of a $50 \times 50 \times 50$ cm block of paraffin with a 45° cone-shaped opening at the front. A neutron dosimeter, providing dose-equivalent rates, was placed behind slabs of shielding material. The shielding material and thickness T were varied over the course of the experiment.

The measured results are shown in Fig. 7-2 (Ueki et al., Fig. 3). The results are reported in terms of an attenuation factor, defined as

$$\text{attenuation factor} = \frac{\text{dose}|_T}{\text{dose}|_{T=0}}. \quad (7-1)$$

No uncertainties in the measured results were reported. For the KRAFTON N2 shielding material, the authors compared measured attenuation factors to results from MCNP2B simulations. Ratios of calculated to experimentally measured results (C/E ratios) ranged from 0.84 to 1.04. Unfortunately, tabulated results were reported only for the KRAFTON material. Results for the other materials were reported in the lin-log plot shown in Fig. 7-2.

7.1.2 Objective

The objective is to (1) use ADVANTG to accelerate continuous-energy MCNP simulations of the graphite shielding experiments performed by Ueki, (2) estimate neutron dose attenuation factors for shield thicknesses of $T = 0, 2, 5, 10, 15, 20, 25, 30,$ and 35 cm, (3) compare the results obtained with and without the ADVANTG-generated weight-window and source biasing parameters, and (4) determine if any statistically significant difference exists between the sets of results.

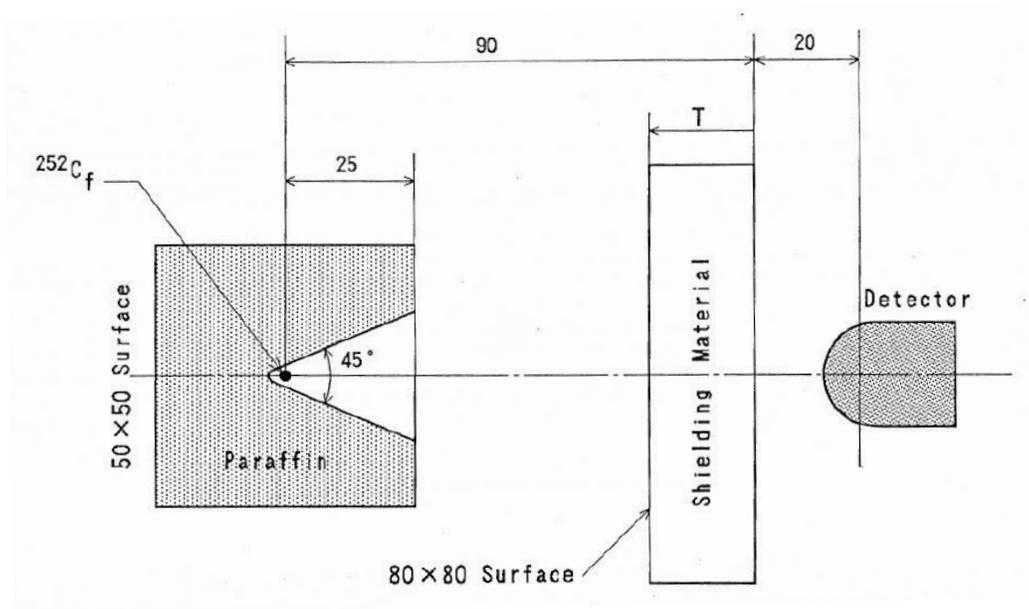


Fig. 7-1. Schematic arrangement of source, shields, and detector (Fig. 2 from Ueki et al.).

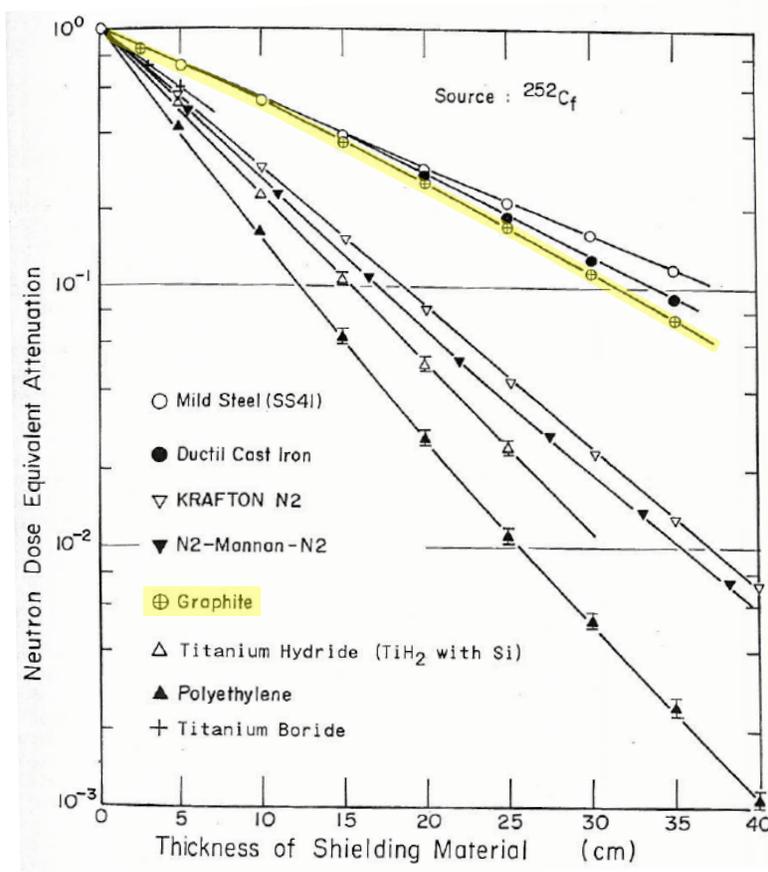


Fig. 7-2. Measured dose attenuation factors (Fig. 3 from Ueki et al.).

7.1.3 MCNP Model and Results

MCNP input files were constructed based on the dimensions shown in Fig. 7-1. Continuous energy ENDF/B-VII.0 cross sections were used for the paraffin and graphite materials, which were modeled at densities of 0.93 and 1.7 g/cm³, respectively. All other regions were modeled as void. The `poly.10t` and `grph.10t` $S(\alpha,\beta)$ tables were used for paraffin and graphite, respectively.

A point neutron source was placed at the center of the paraffin block, as shown in Fig. 7-3. An offset of 0.001 cm was used to avoid placing the source directly on a surface. The source spectrum was modeled as a Watt fission distribution. The neutron dosimeter was idealized as a point detector tally with associated ANSI/ANS-6.1.1 1977 flux-to-dose-rate conversion factors on `DE/DF` cards. The point detector was placed 20 cm behind the back of the shield. To illustrate that the ADVANTG variance reduction parameters are effective at increasing the number of particle tracks in the vicinity of the detector, an F4 tally was defined over a 5 × 5 × 5 cm cube surrounding the detector point.

MCNP simulations were performed on a hex-core Intel Xeon X5690 processor running at 3.47 GHz. A run-time limit of 6 min per case was used. All simulations were performed using only a single core. Tally results for the nine different shield thicknesses are summarized in Table 7-1. All point detector tallies had relative uncertainties of less than 2% and large figures of merit. However, the F4 tally results exhibit much larger uncertainties. This is expected because the volume of the tally cell is small. Though variance reduction parameters are not particularly needed for these simulations, the well-converged point detector results provide a good basis of comparison for the ADVANTG results that will be generated later.

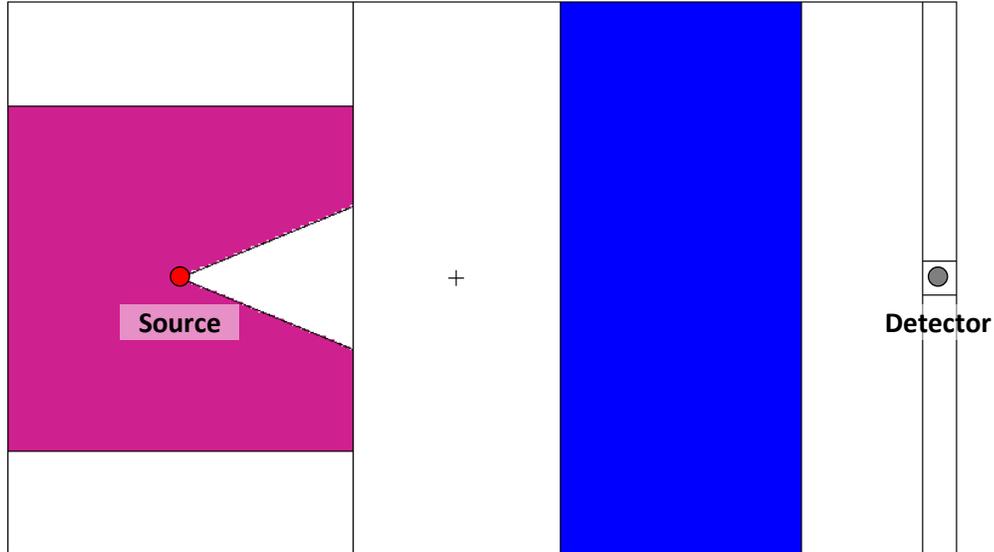


Fig. 7-3. MCNP model of Ueki experiment for the $T = 35$ cm case.

Attenuation factors were calculated based on Eq. (7-1) and are shown in Table 7-2. The table also lists C/E ratios based on experimental values obtained by reading the plot shown in Fig. 7-2. Consequently, the experimental values contain both measurement and reporting uncertainty. For the point detector tally, the C/E ratios range from 1.01 to 1.07, which is roughly comparable to the C/E ratios reported by the authors for the KRAFTON material.

Table 7-1. MCNP5 dose rate results for the Ueki problem

Graphite thickness (cm)	F4 tally		F5 tally	
	Dose rate ($\mu\text{Sv/h}$)	FOM	Dose rate ($\mu\text{Sv/h}$)	FOM
0	454 \pm 8%	25	505 \pm 0.1%	115741
2	449 \pm 8%	26	442 \pm 0.3%	17343
5	359 \pm 9%	20	372 \pm 0.5%	7234
10	245 \pm 10%	16	269 \pm 0.7%	3604
15	182 \pm 12%	11	193 \pm 0.9%	2202
20	133 \pm 14%	8	132 \pm 1.0%	1483
25	89 \pm 17%	6	89 \pm 1.3%	986
30	56 \pm 20%	4	59 \pm 1.6%	668
35	45 \pm 23%	3	40 \pm 1.9%	467

Table 7-2. MCNP5 attenuation factor results for the Ueki problem

Graphite thickness (cm)	F4 tally		F5 tally	
	Attenuation factor	C/E	Attenuation factor	C/E
2	0.989 \pm 11%	1.19	0.876 \pm 0.3%	1.06
5	0.791 \pm 12%	1.10	0.737 \pm 0.5%	1.02
10	0.539 \pm 13%	1.02	0.533 \pm 0.7%	1.01
15	0.401 \pm 15%	1.10	0.382 \pm 0.9%	1.05
20	0.292 \pm 16%	1.15	0.261 \pm 1.1%	1.03
25	0.196 \pm 19%	1.14	0.177 \pm 1.3%	1.03
30	0.123 \pm 21%	1.09	0.116 \pm 1.6%	1.03
35	0.098 \pm 25%	1.32	0.079 \pm 1.9%	1.07

7.1.4 ADVANTG Calculations

ADVANTG input files were created to generate variance reduction parameters using the CADIS method. The input for the $T = 35$ cm case is shown in Fig. 7-4 below. The point detector (tally 5) was defined as the response of interest for the CADIS calculation. ADVANTG defines a point adjoint source at the location of the point detector. This is advantageous, because all point sources are handled with the first-collision treatment in Denovo. This approach greatly reduces ray effects that would otherwise result from small volume sources in nonscattering media. Because the point detector lies at the center of the F4 tally cell, and because the first-collision treatment is desirable in this problem, the F4 tally is not listed on the `mcnp_tallies` card.

A nominally uniform mesh with 2.5-cm-thick voxels was constructed for the Denovo S_N calculation and also for the weight-window parameters. The mesh was defined so as to include the outer boundaries of the paraffin block and the slab shield. Behind the shield, the mesh planes were chosen so as to avoid placing the point adjoint source directly on a mesh boundary (because this can result in nonphysical flux estimates near the point source). The spatial mesh consists of a total of about 61,000 voxels.

```
method                cadis

mcnp_input             ueki35      # MCNP input filename
mcnp_tallies           5          # Tally id(s)
mcnp_material_names    1 paraffin # For visualization output
                     2 graphite

anisn_library          27n19g     # Multigroup library for Denovo
                     # SN calculation

denovo_pn_order        1          # Low-order angular approx is
denovo_quad_num_polar  2          # sufficient for this
denovo_quad_num_azi    2          # scattering-dominated problem

# Mesh is nominally uniform with 2.5cm-thick cells. Planes are selected so
# as to avoid putting a mesh boundary on the detector at (110, 0, 0).

mesh_x                 -25  107.5  112.5  # Coarse mesh boundaries
mesh_x_ints            53      3      # Number of fine meshes per coarse mesh

mesh_y                 -40  -2.5   2.5   40
mesh_y_ints            15      3      15

mesh_z                 -40  -2.5   2.5   40  # Same as y dimension
mesh_z_ints            15      3      15
```

Fig. 7-4. ADVANTG input for the Ueki experiment, $T = 35$ cm case.

For this neutron-only, scattering-dominated problem, a P_1 scattering expansion and a QR quadrature set with two polar and two azimuthal angles per octant (total of 32 angles) were used. These low-order angular treatments are sufficient to capture the attenuation of the neutron flux through the shield and to generate effective variance reduction parameters for this problem. However, this kind of approximation

will generally not produce high-quality deterministic solutions that could be used in place of Monte Carlo results.

To ensure that the spatial mesh was adequate to capture the geometry of the problem, the method card was initially changed to use the `dx` option (see Section 5.2). The discretized model is shown in Fig. 7-5. In the figure, a clip plane at $y = 0$ was used to expose the center of the paraffin block. As discussed in Section 3.4, VisIt applies a material interface reconstruction treatment to material geometry plots by default. Though the image displays an unstructured mesh with sharp material boundaries, the actual geometry used in Denovo is defined on a structured grid with mixed materials in-between clean material regions. After inspecting the material map, the method card was changed back to the `cadis` option to generate variance reduction parameters.

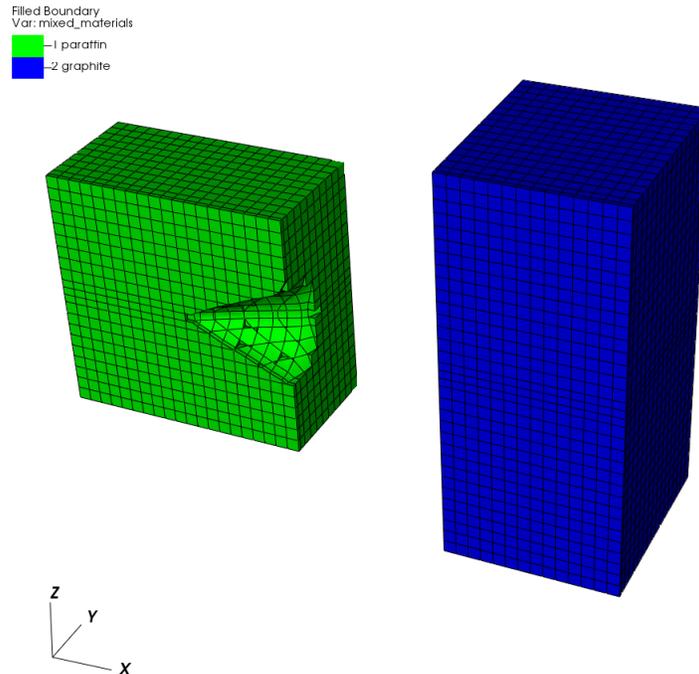


Fig. 7-5. Discretized material map for the Ueki problem.

To obtain effective source biasing parameters, changes were made to the original MCNP input file to partition the SDEF distribution bins. The original SDEF card defined a point source with a Watt fission spectrum, as shown at the top of Fig. 7-6. MCNP provides the capability to bias a continuous distribution with a histogram distribution. ADVANTG provides the capability to calculate importance-weighted biased probabilities for a given set of bins. Before running ADVANTG, an SI card was generated using the `watt.py` script (located in the `share/` directory) and was added to the MCNP input file, as shown at the bottom of Fig. 7-6. The script was used to calculate the boundaries of 100 equiprobable bins up to a maximum energy of 6.5 MeV and 0.5 MeV-width bins from 6.5 to 20 MeV.

```
sdef pos=0.001 0 0 erg=d1
sp1 -3 1.025 2.926
```

original SDEF cards

```
sdef pos=0.001 0 0 erg=d1
sp1 -3 1.025 2.926
c Watt spectrum (a = 1.02500, b = 2.92600)
c from 1.000e-11 to 20.0 MeV
c with 100 equiprobable bins below 6.500 MeV
c with 27 uniform bins above 6.500 MeV
sii 1.000000e-11 9.458907e-02 1.518546e-01 2.009263e-01 2.455698e-01
2.873352e-01 3.270501e-01 3.652217e-01 4.021894e-01 4.381943e-01
4.734157e-01 5.079921e-01 5.420329e-01 5.756272e-01 6.088486e-01
6.417593e-01 6.744124e-01 7.068539e-01 7.391242e-01 7.712594e-01
8.032916e-01 8.352499e-01 8.671611e-01 8.990496e-01 9.309385e-01
9.628489e-01 9.948011e-01 1.026814e+00 1.058907e+00 1.091096e+00
1.123399e+00 1.155832e+00 1.188412e+00 1.221155e+00 1.254076e+00
1.287191e+00 1.320515e+00 1.354066e+00 1.387857e+00 1.421905e+00
1.456226e+00 1.490836e+00 1.525753e+00 1.560992e+00 1.596573e+00
1.632512e+00 1.668829e+00 1.705542e+00 1.742671e+00 1.780238e+00
1.818263e+00 1.856769e+00 1.895781e+00 1.935321e+00 1.975417e+00
2.016095e+00 2.057384e+00 2.099315e+00 2.141919e+00 2.185230e+00
2.229285e+00 2.274122e+00 2.319781e+00 2.366308e+00 2.413747e+00
2.462150e+00 2.511572e+00 2.562069e+00 2.613705e+00 2.666548e+00
2.720673e+00 2.776159e+00 2.833095e+00 2.891577e+00 2.951710e+00
3.013613e+00 3.077412e+00 3.143251e+00 3.211291e+00 3.281708e+00
3.354703e+00 3.430504e+00 3.509368e+00 3.591588e+00 3.677502e+00
3.767504e+00 3.862049e+00 3.961676e+00 4.067024e+00 4.178864e+00
4.298129e+00 4.425973e+00 4.563844e+00 4.713589e+00 4.877624e+00
5.059188e+00 5.262777e+00 5.494888e+00 5.765428e+00 6.090633e+00
6.500000e+00 26i 2.000000e+01
```

modified SDEF cards for ADVANTG

Fig. 7-6. User changes to SDEF cards for the Ueki problem.

ADVANTG calculations were performed for the nine different shield thicknesses. The Denovo run times varied from 39 to 54 seconds per case, while the costs of the other operations performed by ADVANTG were negligible. The total (energy integrated) adjoint scalar flux for the $T = 5$ and 35 cm cases are shown in Figs. 7-7 and 7-8, respectively. The weight-window targets are inversely proportional to the adjoint flux. In this way, particles moving toward the detector are split into multiple tracks, whereas particles traveling toward less important regions (e.g., deep into the paraffin block) are rouletted.

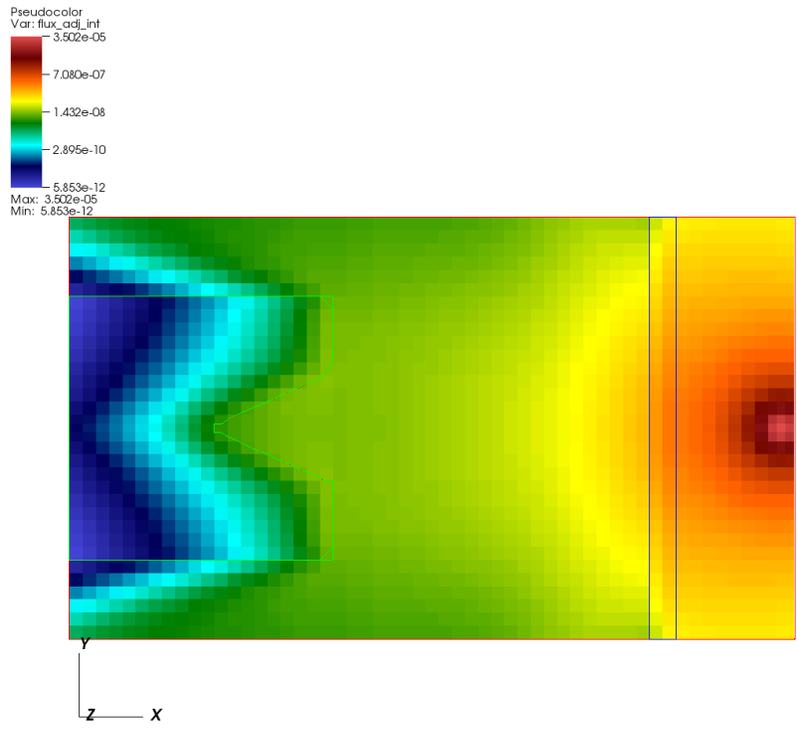


Fig. 7-7. Total adjoint flux for the Ueki problem, $T = 5$ cm case.

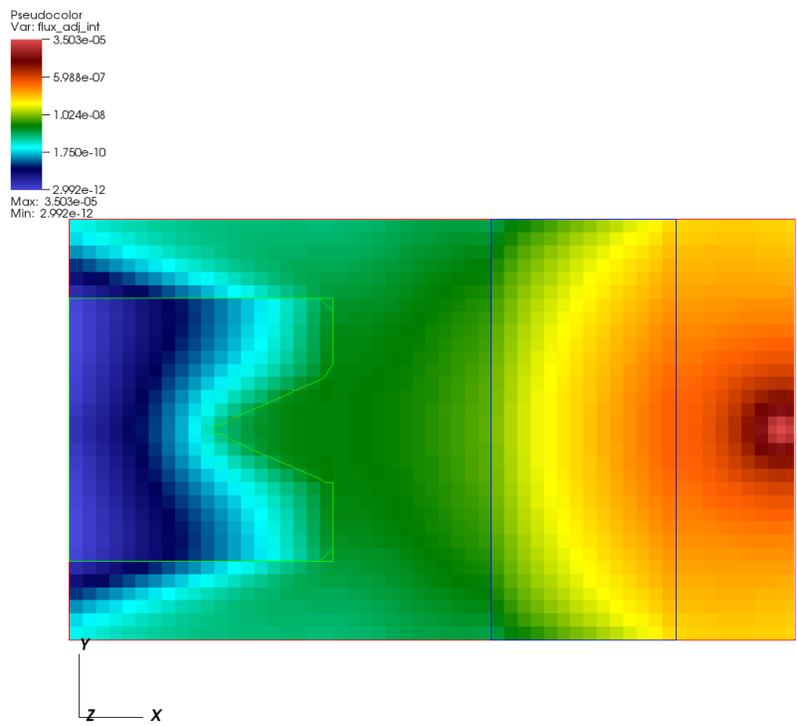


Fig. 7-8. Total adjoint flux for the Ueki problem, $T = 35$ cm case.

Once the CADIS calculation was complete, ADVANTG created a new MCNP input file and a WWINP file containing space- and energy-dependent weight-window bounds in the output/ directory. The changes made to the MCNP input are summarized in Fig. 7-9 below. ADVANTG generated an SB card with importance-weighted biased probabilities for distribution 1 and a WWP card.

```

sdef pos=0.001 0 0 erg=d1
sp1 -3 1.025 2.926
c Watt spectrum (a = 1.02500, b = 2.92600)
si1 1.000000e-11 9.458907e-02 1.518546e-01 2.009263e-01 2.455698e-01
... omitted remainder of si1 card (next 20 lines)
c * added by ADVANTG
sb1 0.00000e+00 5.50735e-04 8.55820e-04 9.66742e-04 9.66741e-04
9.66742e-04 9.66742e-04 9.66740e-04 9.66742e-04 1.75523e-03
1.89621e-03 1.89622e-03 1.89621e-03 1.89622e-03 1.89621e-03
1.89622e-03 1.89622e-03 1.89622e-03 1.89621e-03 1.89622e-03
1.89622e-03 1.89621e-03 1.89622e-03 1.89621e-03 3.18447e-03
3.63250e-03 3.63250e-03 3.63248e-03 3.63258e-03 3.63249e-03
3.63250e-03 3.63246e-03 3.63250e-03 3.63254e-03 3.63250e-03
3.63250e-03 3.63242e-03 3.63261e-03 3.63249e-03 3.63250e-03
6.04662e-03 6.10165e-03 6.10181e-03 6.10160e-03 6.10183e-03
6.10171e-03 6.10182e-03 6.10173e-03 6.10166e-03 6.10180e-03
6.10169e-03 9.41598e-03 1.03452e-02 1.03448e-02 1.03450e-02
1.03449e-02 1.03449e-02 1.03451e-02 1.03450e-02 1.03449e-02
1.03450e-02 1.03450e-02 1.03448e-02 1.03452e-02 1.03448e-02
1.03449e-02 1.03452e-02 1.03449e-02 1.03449e-02 1.03449e-02
1.03451e-02 1.03449e-02 1.03450e-02 1.03450e-02 1.03448e-02
1.05712e-02 1.73959e-02 1.73959e-02 1.73962e-02 1.73960e-02
1.73958e-02 1.73960e-02 1.73961e-02 1.73960e-02 1.73958e-02
1.73961e-02 1.73960e-02 1.73960e-02 1.73959e-02 1.73961e-02
1.73960e-02 1.73959e-02 1.73960e-02 1.73960e-02 1.73961e-02
1.73960e-02 1.73960e-02 1.73960e-02 1.73960e-02 1.73960e-02
2.45877e-02 3.99747e-02 2.88561e-02 2.07155e-02 1.47975e-02
1.05223e-02 7.45134e-03 5.25653e-03 3.69516e-03 2.58909e-03
1.80859e-03 1.25979e-03 8.75187e-04 6.06482e-04 4.19288e-04
2.89229e-04 1.99094e-04 1.36776e-04 9.37867e-05 6.41934e-05
4.38627e-05 2.99220e-05 2.03802e-05 1.38604e-05 9.41284e-06
6.38366e-06 4.32361e-06 2.92464e-06
c * added by ADVANTG
wwp:n 5.0 j 100 j -1 0 5.600880578e-11

```

Fig. 7-9. ADVANTG changes to MCNP input for Ueki problem, $T = 35$ cm case.

As discussed in Section 3.8, the WNORM value (seventh entry) on the WWP:n card is the estimated value of R , normalized to a unit source strength based on the discrete ordinates calculation. For the $T = 35$ cm case, this value is 5.6×10^{-11} rem/h per n/s. Multiplying this value by the source strength of 4.05×10^7 n/s yields an estimate of R of $22.7 \mu\text{Sv/h}$. This dose rate is about a factor of two lower than the MCNP F5 tally result. The magnitude of this difference is expected given the parameters selected for the deterministic calculation. Nonetheless, the deterministic solution is of sufficient quality to generate effective variance reduction parameters, as will be shown in the next subsection.

The ratio of biased to unbiased source energy bin probabilities is plotted in Fig. 7-10 for the $T = 35$ cm case. The source biasing reduces the probability of sampling energies below about 2 MeV, with a minimum ratio of 0.056. The probability of sampling high-energy source particles is increased significantly, because these particles have a much greater probability of penetrating the shield and reaching the detector. Note that in the MCNP simulation, source particle weights are calculated as the ratio of the unbiased to biased probability (reciprocal of the curve plotted below) to preserve the tally mean value.

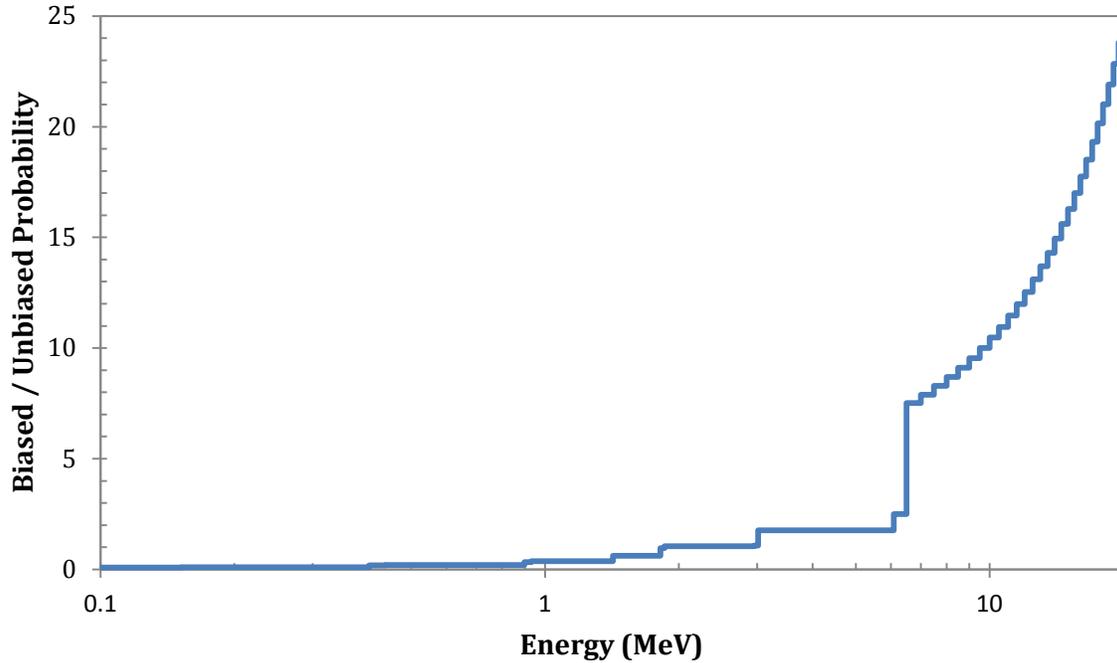


Fig. 7-10. Ratio of biased to unbiased source bin probabilities, $T = 35$ cm case.

7.1.5 Results

The ADVANTG-generated variance reduction parameters were used in MCNP simulations of the nine cases. To perform an equal-time comparison, the run-time of each simulation was limited to five minutes so that the sum of the Denovo and MCNP run times did not exceed the six minutes of the initial MCNP calculations. The ADVANTG-based dose rate and attenuation factor results are shown in Tables 7-3 and 7-4, respectively. The MCNP and ADVANTG dose rates results for the F4 and F5 tallies are compared in Tables 7-5 and 7-6. The FOM shown in the tables for the ADVANTG results was adjusted to account for Denovo run time, according to

$$\text{adjusted FOM} = \frac{1}{R^2(T_{MC} + T_{ADVANTG})}, \quad (7-2)$$

where R is the tally relative error, T_{MC} is the Monte Carlo run time in minutes, and $T_{ADVANTG}$ is the ADVANTG run time. (Generally, the ADVANTG run time is dominated by the S_N calculation.) This adjusted FOM can be used to determine whether the ADVANTG-generated parameters are worth the time

that was required to generate them. This metric can, of course, be abused by making T_{MC} larger than what is actually required in practical applications.

For this problem, the ADVANTG parameters are successful at significantly reducing the variance of both the F4 and F5 tally results. The FOMs are 15 to 600 times higher for the F4 tally and 15 to 60 times higher for the F5 tally, when accounting for the ADVANTG run time. Moreover, no statistically significant differences are observed between the MCNP results with and without the ADVANTG-generated parameters. The C/E ratios for the ADVANTG-based attenuation factor results range from 1.04 to 1.09 and 1.02 to 1.08 for the F4 and F5 tallies, respectively, whereas only the MCNP point detector results showed agreement at this level.

Table 7-3. ADVANTG-based dose rate results for the Ueki problem

Graphite thickness (cm)	F4 tally		F5 tally	
	Dose rate ($\mu\text{Sv/h}$)	FOM	Dose rate ($\mu\text{Sv/h}$)	FOM
0	495 \pm 2.2%	376	504 \pm 0.03%	1949318
2	438 \pm 1.4%	853	441 \pm 0.04%	1105217
5	370 \pm 1.0%	1831	371 \pm 0.07%	358877
10	270 \pm 0.8%	2854	271 \pm 0.11%	143522
15	189 \pm 0.8%	3065	192 \pm 0.16%	67349
20	132 \pm 0.8%	2970	132 \pm 0.19%	47528
25	89 \pm 0.8%	2740	90 \pm 0.22%	35328
30	59 \pm 0.9%	2292	60 \pm 0.26%	25073
35	40 \pm 0.9%	1922	40 \pm 0.33%	15595

Table 7-4. ADVANTG-based attenuation factor results for the Ueki problem

Graphite thickness (cm)	F4 tally		F5 tally	
	Attenuation factor	C/E	Attenuation factor	C/E
2	0.884 \pm 2.6%	1.07	0.876 \pm 0.1%	1.06
5	0.747 \pm 2.4%	1.04	0.735 \pm 0.1%	1.02
10	0.546 \pm 2.3%	1.04	0.538 \pm 0.1%	1.02
15	0.382 \pm 2.3%	1.05	0.381 \pm 0.2%	1.04
20	0.266 \pm 2.3%	1.05	0.262 \pm 0.2%	1.04
25	0.180 \pm 2.3%	1.05	0.178 \pm 0.2%	1.04
30	0.120 \pm 2.3%	1.06	0.120 \pm 0.3%	1.06
35	0.081 \pm 2.4%	1.09	0.080 \pm 0.3%	1.08

Table 7-5. Comparison of ADVANTG and MCNP5 F4 tally results

Graphite thickness (cm)	MCNP		ADVANTG	
	Dose rate ($\mu\text{Sv/h}$)	FOM	Dose rate ($\mu\text{Sv/h}$)	FOM
0	454 \pm 8%	25	495 \pm 2.2%	376
2	449 \pm 8%	26	438 \pm 1.4%	853
5	359 \pm 9%	20	370 \pm 1.0%	1831
10	245 \pm 10%	16	270 \pm 0.8%	2854
15	182 \pm 12%	11	189 \pm 0.8%	3065
20	133 \pm 14%	8	132 \pm 0.8%	2970
25	89 \pm 17%	6	89 \pm 0.8%	2740
30	56 \pm 20%	4	59 \pm 0.9%	2292
35	45 \pm 23%	3	40 \pm 0.9%	1922

Table 7-6. Comparison of ADVANTG and MCNP5 F5 tally results

Graphite thickness (cm)	MCNP		ADVANTG	
	Dose rate ($\mu\text{Sv/h}$)	FOM	Dose rate ($\mu\text{Sv/h}$)	FOM
0	505 \pm 0.1%	115741	504 \pm 0.03%	1949318
2	442 \pm 0.3%	17343	441 \pm 0.04%	1105217
5	372 \pm 0.5%	7234	371 \pm 0.07%	358877
10	269 \pm 0.7%	3604	271 \pm 0.11%	143522
15	193 \pm 0.9%	2202	192 \pm 0.16%	67349
20	132 \pm 1.0%	1483	132 \pm 0.19%	47528
25	89 \pm 1.3%	986	90 \pm 0.22%	35328
30	59 \pm 1.6%	668	60 \pm 0.26%	25073
35	40 \pm 1.9%	467	40 \pm 0.33%	15595

7.2 SIMPLIFIED PORTAL MONITOR

7.2.1 Background

Portal monitors are large detector panels used to screen cargos for illicit radioactive materials at ports of entry and weigh stations. The portal monitoring scenario described in the example problem section of Wagner et al. (2014) models a cargo container holding a ^{133}Ba point source and an array of large blocks of homogenized iron and polyethylene that partially shield the source. The geometry of the problem is shown in Fig. 7-11. The detector panel is greatly simplified and consists only of the four large sodium-iodide (NaI) crystals that are located in front of the container. The point source is located in a streaming pathway at the geometric center of the container, as shown in Fig. 7-12.

7.2.2 Objective

The objective is to: (1) use ADVANTG to accelerate a continuous-energy MCNP photon simulation of the simplified portal monitoring scenario, (2) estimate the pulse-height spectrum integrated over the four large NaI crystals resulting from the ^{133}Ba point source inside the cargo container, (3) use the FW-CADIS method to obtain relatively uniform uncertainties over 1 keV pulse-height (energy) bins, (4) compare the results obtained with and without the ADVANTG-generated weight-window and source biasing parameters, and (5) determine if any statistically significant difference exists between the sets of results.

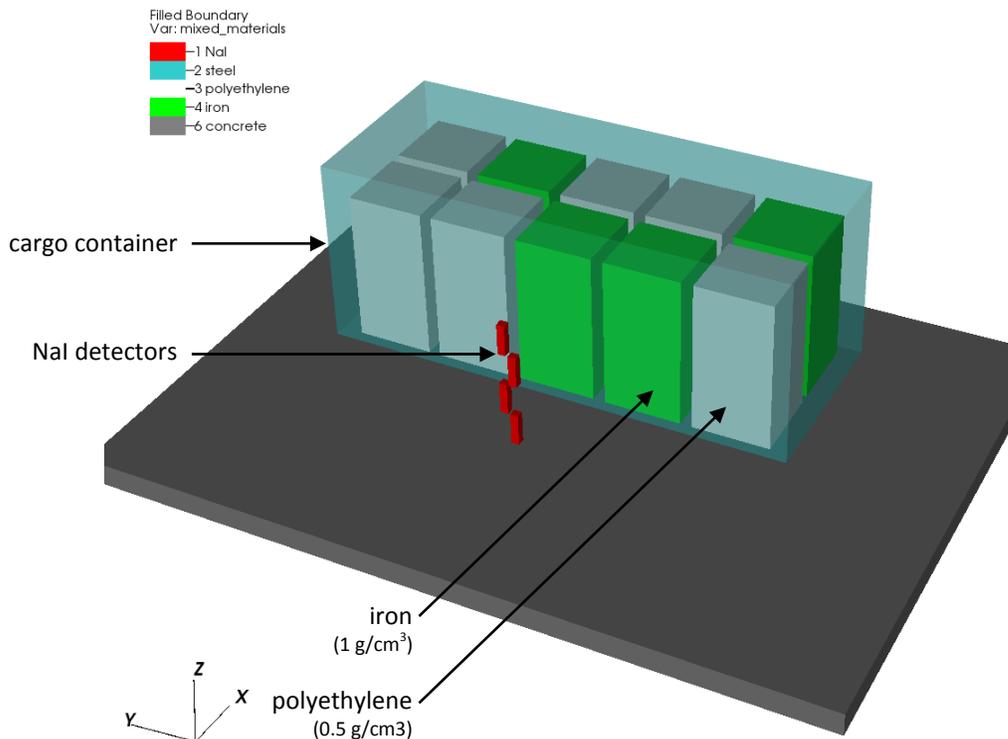


Fig. 7-11. Simplified portal problem.

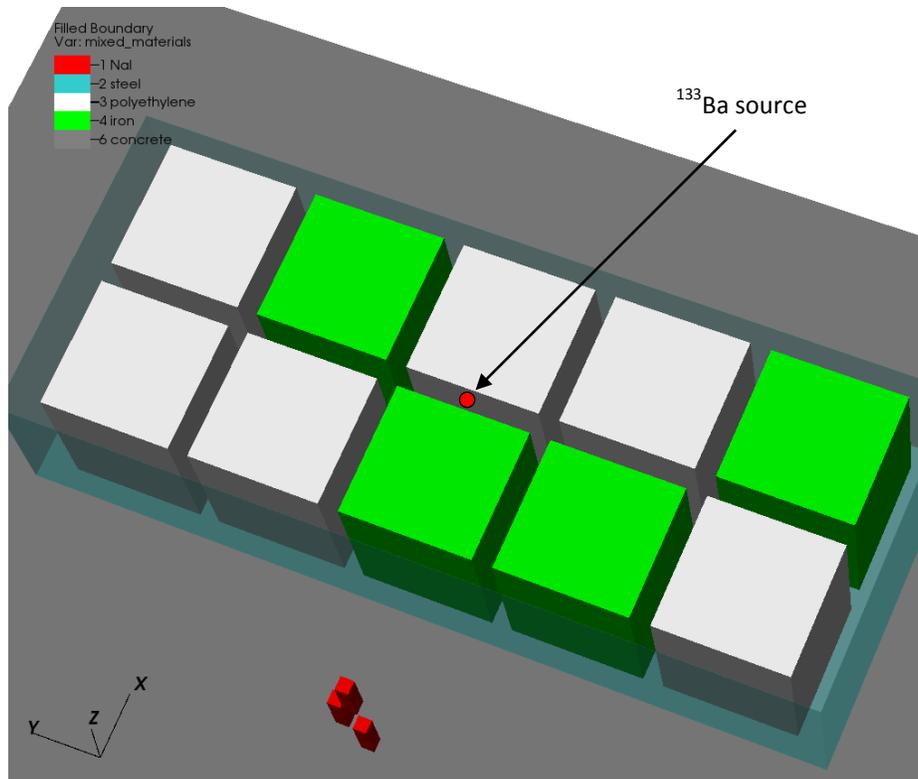


Fig. 7-12. Source location in portal problem.

7.2.3 ADVANTG Calculations

The ADVANTG input file for the simplified portal problem is shown in Fig. 7-13. The response of interest for the FW-CADIS calculation is the F8 pulse height tally. The tally has a single cell bin, which includes the four NaI cells, and multiple pulse-height bins that range from 1 to 400 keV in 1 keV widths. (The zero bin and epsilon bin recommended by the MCNP manual are also included.) The default response weighting treatment was turned off to obtain relatively uniform uncertainties across all pulse-height bins.

A nonuniform mesh that captures all of the material boundaries was constructed for the Denovo forward and adjoint S_N calculations. Within the cargo container, the voxels are nominally 10 cm thick. Additional mesh planes, parallel to the x -axis, were added to the gaps between the homogenized iron and polyethylene blocks. The entire mesh consists of a total of about 600,000 voxels. For this photon-only problem where streaming and scattering effects are important, a P_3 scattering-angle expansion and QR quadrature set with four polar and four azimuthal angles per octant (total of 128 angles) were used. This level of angular approximation is sufficient for generating variance reduction parameters for this problem.

The highest energy emission line of ^{133}Ba is 383.8 keV. Because weight-window bounds above this energy will not be used in the Monte Carlo simulation, the `denovo_first_group` card was used to set the first (highest energy) group of the discrete ordinates calculation to group number 41. (In the 27n19g library, group 41 has an upper energy of 400 keV, as shown in Table A-1 of Appendix A.) This reduces the overall run time of the forward and adjoint deterministic calculations performed in the FW-CADIS method.

```

method                fwcadis
fwcadis_response_weighting  false      # Turn off response weighting

mcnp_input            cargo
mcnp_tallies          8
mcnp_material_names  101  NaI
                    102  steel
                    103  polyethylene
                    104  iron
                    105  air
                    106  concrete

anisn_library         27n19g

denovo_pn_order       3      # Use higher-order angular approx
denovo_quad_num_polar  4      # for this photon-only problem
denovo_quad_num_azi   4

denovo_first_group    41     # Highest Ba-133 line is 383.8 keV

mesh_x               -182.8800   0.0000   10.1600   12.1600   86.5600
                    90.5403   93.5268   95.7677   97.4490   98.7105
                    99.6571  100.3670  100.9000  101.3000  102.2000
                    102.6000  103.1330  103.8430  104.7890  106.0510
                    107.7320  109.9730  112.9600  116.9400  216.0000
                    231.0400  330.1000  335.1130  340.1270  345.1400
                    345.4400  386.0360  406.3330  467.2270  487.5240
                    528.1200

mesh_x_ints          9          5          1          16
                    1          1          1          1          1
                    1          1          1          1          3
                    1          1          1          1          1
                    1          1          1          1          10
                    5          10         1          1          1
                    1          2          1          3          1
                    2

```

continued on next page

As with the Ueki problem, the dx option was initially used to study the discretized material map shown in Figs. 7-11 and 7-12. In this problem, all material boundaries are orthogonal to one of the spatial axes, so it is possible to avoid material mixing altogether.

The original SDEF cards defined a point source with 13 discrete source energies. In this case, no partitioning of the SDEF distribution bins is possible and hence no changes need to be made to the original MCNP input file before running ADVANTG.

continued from previous page

mesh_y	-487.4800	-446.8840	-426.5870	-365.6930	-345.3960
	-304.8000	-304.5000	-277.7300	-178.6700	-173.6570
	-168.6430	-163.6300	-64.5700	-49.5300	-40.1633
	-30.7967	-21.4300	-1.2700	1.2700	21.4300
	30.7967	40.1633	49.5300	64.5700	163.6300
	168.6430	173.6570	178.6700	277.7300	304.5000
	304.8000	345.3960	365.6930	426.5870	446.8840
	487.4800				
mesh_y_ints		2	1	3	1
	2	1	2	10	1
	1	1	10	5	1
	1	1	10	2	10
	1	1	1	5	10
	1	1	1	10	2
	1	2	1	3	1
	2				
mesh_z	-30.4800	-19.8951	-12.7517	-7.9309	-4.6774
	-2.4818	-1.0000	0.0000	2.4870	4.6983
	7.9866	12.8763	20.1475	30.9600	43.5443
	50.9236	55.2507	57.7880	59.2759	60.1484
	60.6600	61.5600	62.1739	63.4300	66.0004
	71.2600	81.3733	91.4867	101.6000	223.5200
	259.3800	304.5000	304.8000	487.6800	
mesh_z_ints		1	1	1	1
	1	1	1	1	1
	1	1	1	1	1
	1	1	1	1	1
	1	3	1	1	1
	1	1	1	1	12
	4	5	1	9	

Fig. 7-13. ADVANTG input for portal problem.

ADVANTG was executed to generate variance reduction parameters using the FW-CADIS method. This requires both a forward and an adjoint Denovo calculation, which took 6.7 and 4.0 min, respectively. The other tasks performed by ADVANTG required only a few seconds.

The total forward and adjoint scalar fluxes are shown at the height of the point source in Fig. 7-14. In the figure, the left side image contains an inset image of the material map that shows the homogenized iron and polyethylene blocks as green and white, respectively. The fact that the iron material is more effective at attenuating the photon flux can be clearly observed in the forward flux map. Ray effects are visible in the adjoint flux map in the region between the detectors and the front of the cargo container. This region has a high importance because of its proximity to the detectors. However, the weight-window treatment is almost never applied in this area, because it contains only air. (MCNP applies weight windows only at surface crossings, collisions, and every mean free path of travel.) For this reason, and because the importance map within the cargo container appears to be reasonable, no further refinements were made to the deterministic calculations.

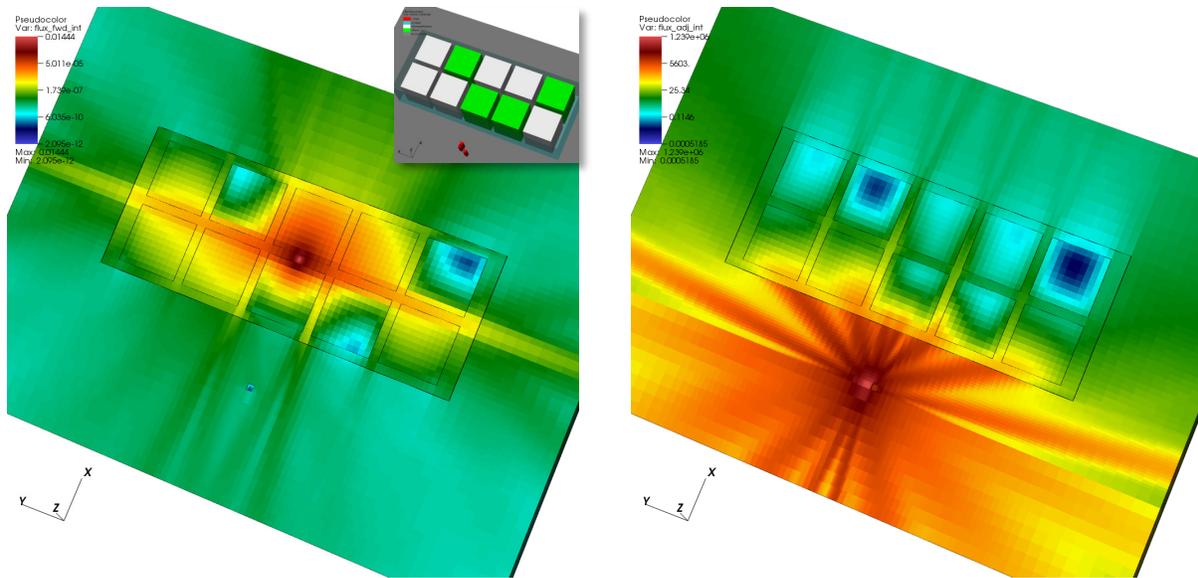


Fig. 7-14. Denovo forward (left) and adjoint (right) total fluxes for the portal problem. The fluxes are shown in the x - y plane at the source height.

ADVANTG created a new MCNP input file and a WWINP file. The changes made to the MCNP input are summarized in Fig. 7-15. ADVANTG generated an SB card with importance-weighted biased probabilities for distribution 1 and a WWP card. For this case, ADVANTG also generated a VAR card to turn off Russian roulette; this is required for MCNP to perform correct variance reduction with pulse-height tallies.

```

sdef pos 223.52 0 182.88 erg=d1
si1 L 4.619900e-03 3.062500e-02 3.097300e-02 3.495300e-02
      5.316220e-02 7.961420e-02 8.099790e-02 1.606120e-01
      2.232368e-01 2.763989e-01 3.028508e-01 3.560129e-01
      3.838485e-01
sp1 6.231324e-02 1.264485e-01 2.335836e-01 8.454194e-02
     7.798265e-03 9.656731e-03 1.200714e-01 2.332192e-03
     1.639822e-03 2.609139e-02 6.683186e-02 2.261133e-01
     3.257780e-02
c * added by ADVANTG
sb1 6.39933e-07 1.29858e-06 2.39881e-06 8.68213e-07
     2.68898e-03 3.32981e-03 4.14027e-02 3.25938e-03
     3.37495e-03 5.36992e-02 1.83182e-01 6.19764e-01
     8.92939e-02
c * added by ADVANTG
wvp:p 5.0 j 100 j -1 0 4.005146570e-01
var rr=off

```

Fig. 7-15. ADVANTG changes to SDEF cards for the portal problem.

The biasing of the source emission lines is shown in Table 7-7 below. Not surprisingly, the ratio of the biased to unbiased probability increases strongly with energy. The probabilities reflect the fact that the highest energy lines have the best chance of contributing to the pulse height tally over the detector cells. In an analog calculation, about 50% of the source particles would be sampled with energies below 50 keV. With the ADVANTG biased source, only about 5 in every 10^6 source particles will be sampled from these lines, because they have a negligible impact on the tally results.

Table 7-7. ADVANTG source energy biasing

Energy (keV)	Unbiased probability	Biased probability	Biased / unbiased
383.8	0.033	0.089	
356.0	0.226	0.620	2.741
302.9	0.067	0.182	
276.4	0.026	0.054	2.058
223.2	0.002	0.003	
160.6	0.002	0.003	1.398
81.0	0.120	0.041	
79.6	0.010	0.003	0.345
53.2	0.008	0.003	
35.0	0.085	8.68E-07	
31.0	0.234	2.40E-06	1.03E-05
30.6	0.126	1.30E-06	
4.6	0.062	6.40E-07	

7.2.4 Results

MCNP simulations of the simplified portal problem with and without the ADVANTG-generated variance reduction parameters were performed. To obtain reasonably well-converged results, the run time limits were set to 95 and 96 CPU hours, respectively. The pulse height tally results and relative uncertainties are displayed in Figs. 7-16 and 7-17. The fraction of pulse-height bins with a relative uncertainty level at or below a given level is plotted in Fig. 7-18.

The FW-CADIS variance reduction parameters have a dramatic impact on the precision of the pulse-height tally results for this problem. Relatively uniform statistical uncertainties are obtained across all pulse-height bins. Without variance reduction, 36, 57, and 67% of the pulse-height bins have relative errors less than or equal to 5, 10, and 15%, respectively. With the FW-CADIS variance reduction parameters, 92, 93, and 100% of the bins have uncertainties at or below these levels.

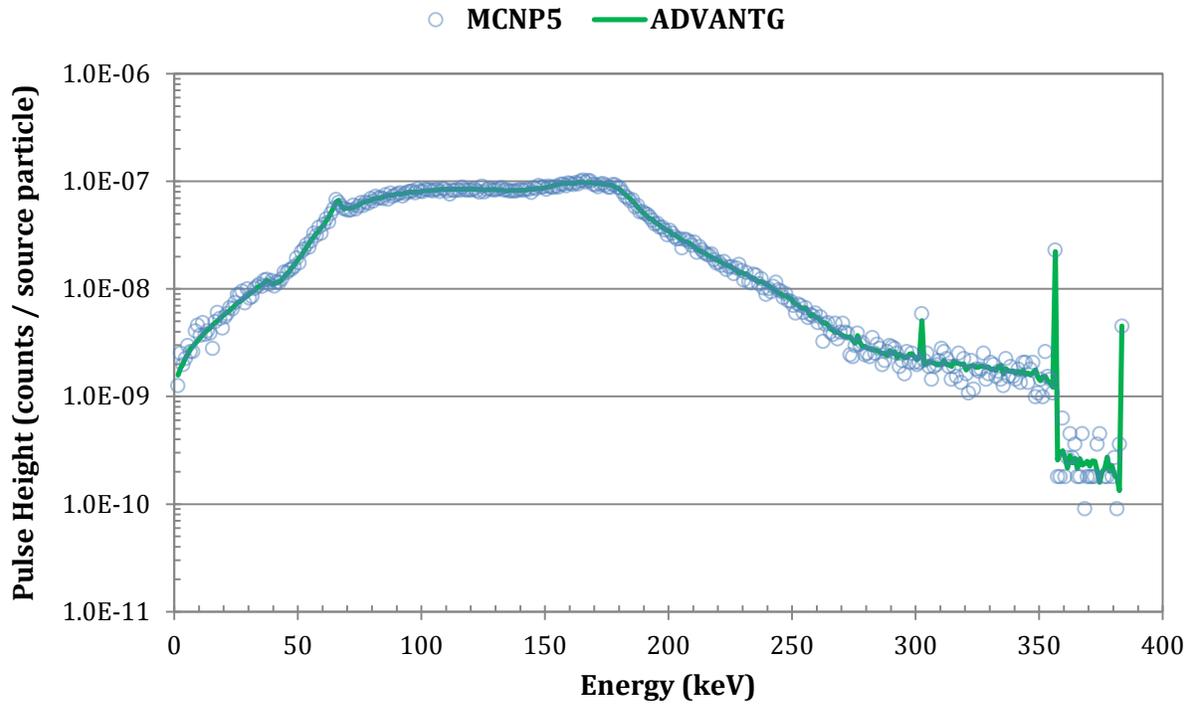


Fig. 7-16. Pulse height spectra for the portal problem.

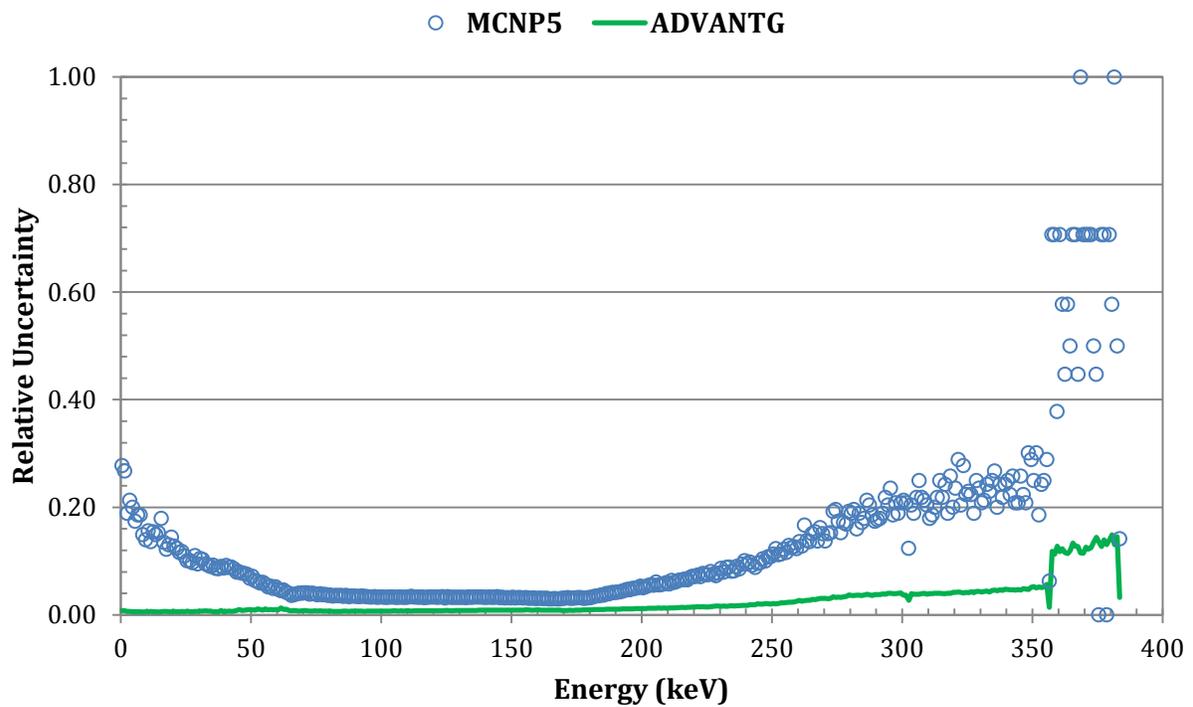


Fig. 7-17. Pulse-height relative uncertainties for the portal problem.

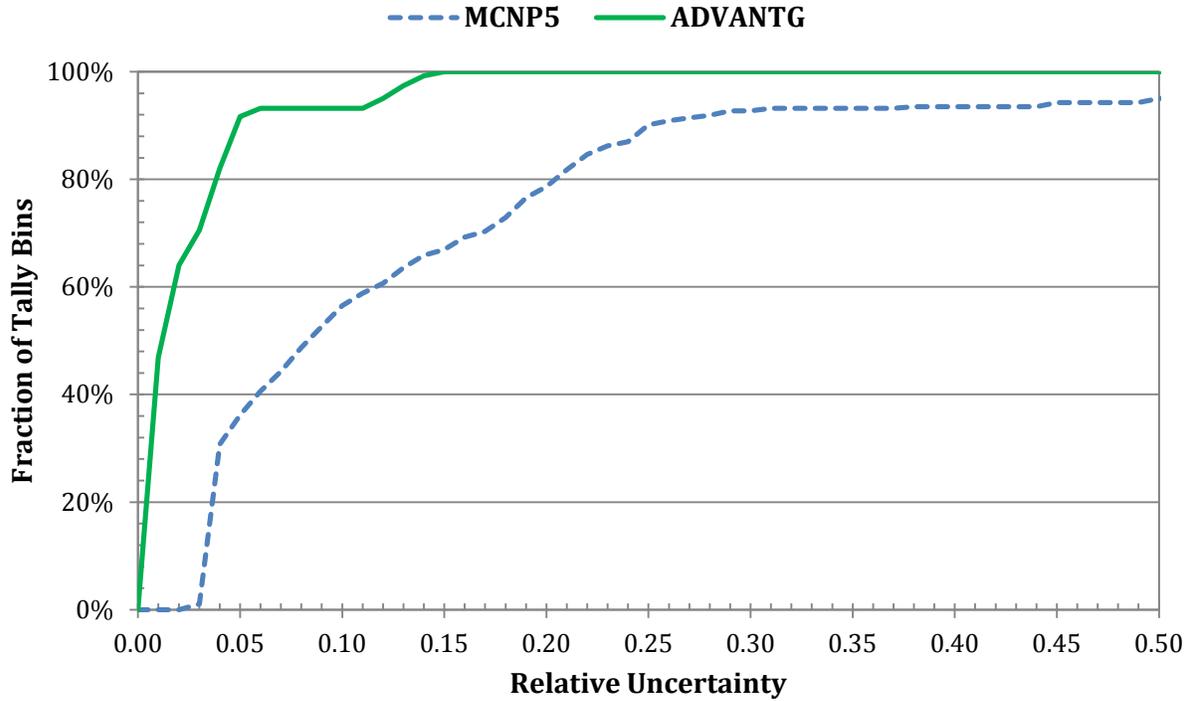


Fig. 7-18. Fraction of tally bins with less than a given uncertainty for the portal problem.

The difference between the pulse-height results estimated with and without the ADVANTG-based parameters was calculated for each bin according to

$$\text{Difference} = \frac{|ADVANTG - MCNP|}{\sqrt{\sigma_{ADVANTG}^2 + \sigma_{MCNP}^2}} \quad (7-3)$$

The distribution of differences for this problem is shown in Table 7-8 below. The differences are approximately normally distributed; this supports the assertion that both sets of pulse-height results were drawn from the same underlying distribution.

Table 7-8. Distribution of differences for the simplified portal problem

Difference level	Fraction of bins within (%)	Normal distribution (%)
< 1σ	68.06	68.27
< 2σ	95.03	95.45
< 3σ	99.48	99.73
< 4σ	100.00	99.99

7.3 JAPAN POWER DEMONSTRATION REACTOR

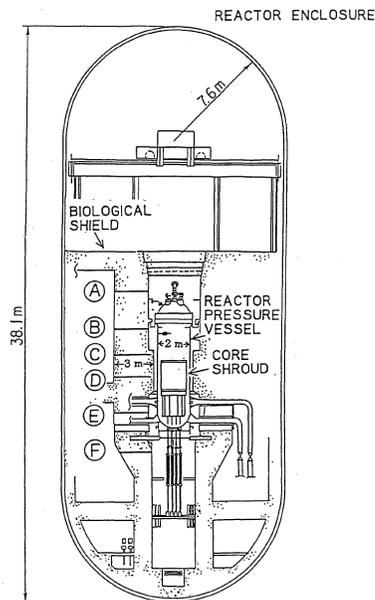
7.3.1 Background

The Japan Power Demonstration Reactor (JPDR) was a prototype boiling water reactor that operated from 1963 to 1976. It was the first nuclear reactor to produce electricity in Japan and provided data and experience for later commercial reactors. It also provided a test bed for reactor decommissioning. A program to decommission the reactor began in 1981 and was completed by the mid-1990s. A schematic of the reactor enclosure and associated specifications are shown in Fig. 7-19 (Fig. 1 and Table A.1 from Sukegawa et al. 1993).

A fundamental task in planning for a decommissioning effort is estimating the radionuclide inventory in structural materials and equipment within the reactor building. Sukegawa et al. describes a 2-D, cylindrical (r, z) benchmark model of the JPDR. The benchmark was developed by the Japan Atomic Energy Research Institute to verify radionuclide inventory estimation techniques. This problem is challenging because even highly attenuated fluxes deep within concrete can contribute significantly to the build-up of ^{60}Co (from ^{59}Co impurities in steel rebar) and other radionuclides over many years of operation.

7.3.2 Objective

The objective is to: (1) use ADVANTG to accelerate a continuous-energy MCNP neutron simulation of the JPDR model, (2) estimate the energy-integrated flux distribution throughout the model with relatively uniform uncertainties over a rectangular mesh tally, and (3) compare the results obtained with and without the ADVANTG-generated weight-window and source biasing parameters. For the purpose of this example problem, the source will be modeled as emitting particles uniformly from the cylinder that encompasses the fuel region.



Type of reactor	BWR
Thermal power	90 MWt (45 MWt initially)
Core assembly	
core diameter	130 cm
active fuel length	147 cm
number of fuel assemblies	72
Pressure vessel	
material	ASTM-A302-56 GrB
inner diameter	2.1 m
height	8.1 m
thickness	7 cm
Biological shield	
material	reinforced concrete
thickness	1.5 to 3 m
inner diameter	2.7 m
Reactor enclosure	
inner diameter	15 m
height	38 m

Fig. 7-19. JPDR reactor enclosure and specifications (Fig. 1 and Table A.1 from Sukegawa et al.).

7.3.3 MCNP Model and Results

The geometry of the benchmark model is shown in Fig. 7-20 (Figs. A.5[1] – [3] from Sukegawa et al.). Material compositions and temperatures are given in tables and in text, respectively, in the reference. MCNP input files were constructed based on all of this information. Continuous energy ENDF/B-VII.0 cross sections at 293, 600, 900, and 1200 K were used. The `lwtr.16t S(α,β)` tables were used for all materials that contained a significant fraction of water and for the concrete material. The geometry of the MCNP model is shown in Fig. 7-21.

The source distribution defined by the benchmark problem models the nonuniform radial and axial flux profiles that existed within the reactor core. For the purpose of this example, a simplified source description was used to distribute particles uniformly within the cylinder that encompasses the fuel region (64.79 cm radius and 146.7 cm height). The source spectrum was modeled as a Watt fission distribution with default parameters. A rectangular mesh tally with 10.7-cm-thick cells was used to tally the spatial distribution of the total flux. The mesh tally contained a total of about 500,000 voxels.

An MCNP simulation was performed on a hex-core desktop system with a run-time limit of 720 min. The simulation was performed in an hour of wall-clock time using 12 threads. Plots of the total flux per source neutron and associated relative uncertainties are shown in Fig. 7-22. In constructing the relative uncertainty plot, uncertainties of zero were set to one so that the voxels that received no tally scores do not appear to be highly converged. As is typical of conventional Monte Carlo simulations, low uncertainties were obtained near the source region. The mesh tally voxels far from the source received very few or no contributions at all. A very long run time would be required to obtain accurate estimates of the flux deep within the concrete.

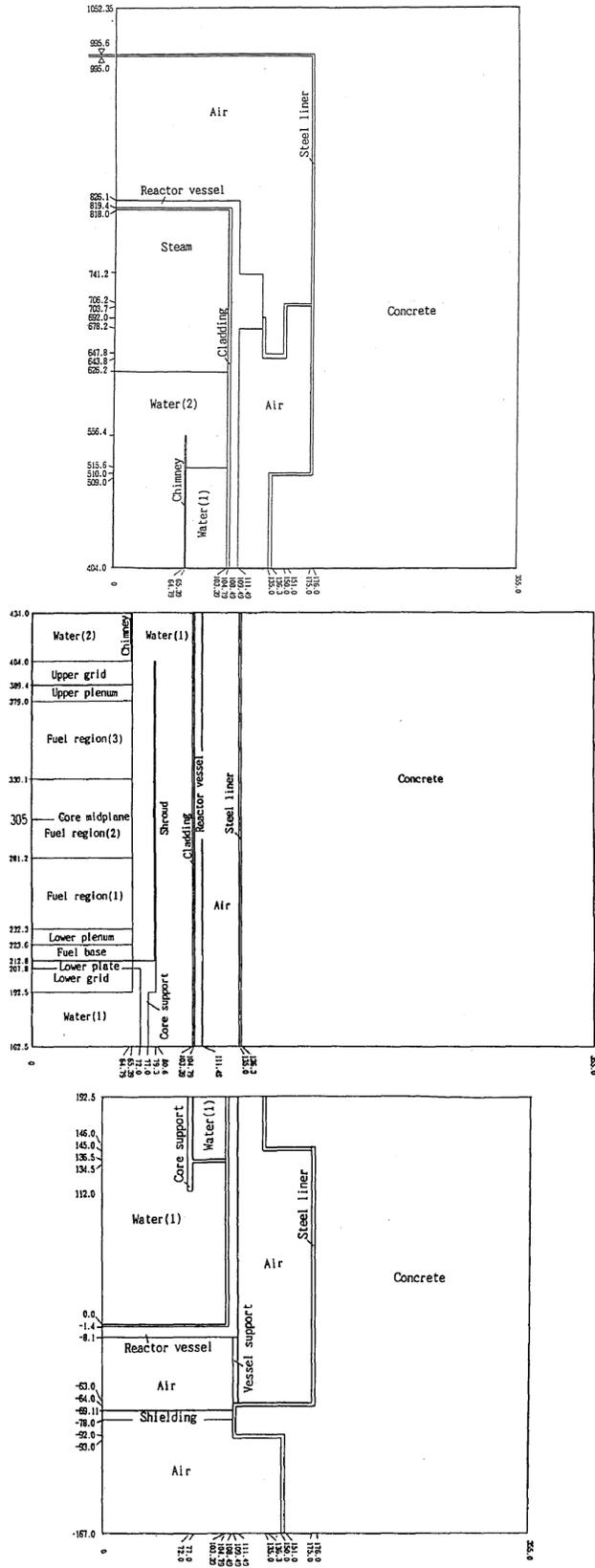


Fig. 7-20. JPDR 2-D (r, z) benchmark geometry (Figs. A.5[1] - [3] from Sukegawa et al.).

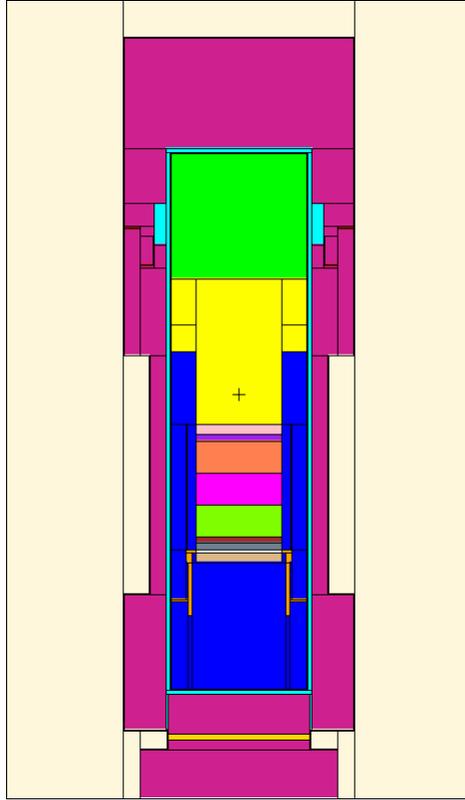


Fig. 7-21. JPDR MCNP model.

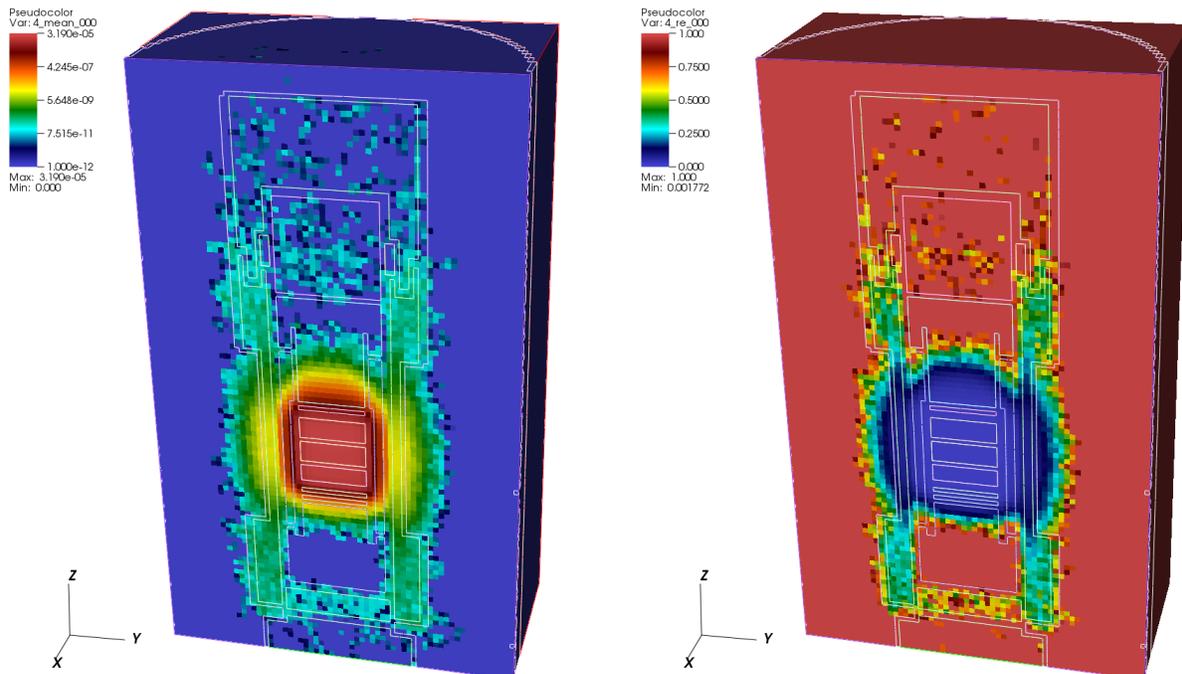


Fig. 7-22. MCNP total flux (left) and relative uncertainty (right) for the JPDR problem.

7.3.4 ADVANTG Calculations

The ADVANTG input file for the JPDR problem is shown in Fig. 7-23. The response of interest for the FW-CADIS calculation is the mesh tally defined by the FMESH4:n card in the MCNP input file. To obtain relatively uniform statistical uncertainties throughout the voxels of the mesh tally, the global weighting option was used. Because the objective of this example is to estimate total fluxes, the default response-weighting treatment was left on.

A uniform mesh with 8.5-cm-thick voxels was constructed for Denovo forward and adjoint S_N calculations. The entire mesh contains a total of about one million voxels. A P_1 scattering expansion and the default QR quadrature set with four polar and four azimuthal angles per octant were used. Because the problem is relatively large, the Denovo calculations were performed in parallel on four cores.

```
method                fwcadis
fwcadis_spatial_treatment  global

mcnp_input            jpdr
mcnp_tallies          4          # Mesh tally
mcnp_material_names  1  air          26  "fuel base"
                    11  water1       27  "lower plate"
                    12  water2       28  "lower grid"
                    13  steam        31  fuel1
                    21  vessel       32  fuel2
                    22  shroud       33  fuel3
                    23  "upper grid" 41  concrete
                    24  "upper plenum" 42  "steel liner"
                    25  "lower plenum" 43  "lower shield"

anisn_library         bplus

denovo_pn_order       1

denovo_x_blocks       2          # Parallel job on 4 cores
denovo_y_blocks       2
denovo_z_blocks       3          # Use 3 pipelining blocks

# Uniform mesh with 8.5cm-thick cells.
mesh_x                -355  355
mesh_x_ints           83

mesh_y                -355  355          # Same as x dimension
mesh_y_ints           83

mesh_z                -167  1052.35
mesh_z_ints           144
```

Fig. 7-23. ADVANTG input for the JPDR problem.

As in previous examples, the dx option was initially used to study the discretized material map, shown in Fig. 7-24. For this problem, the 8.5-cm uniform mesh captures most of the details of the geometry model.

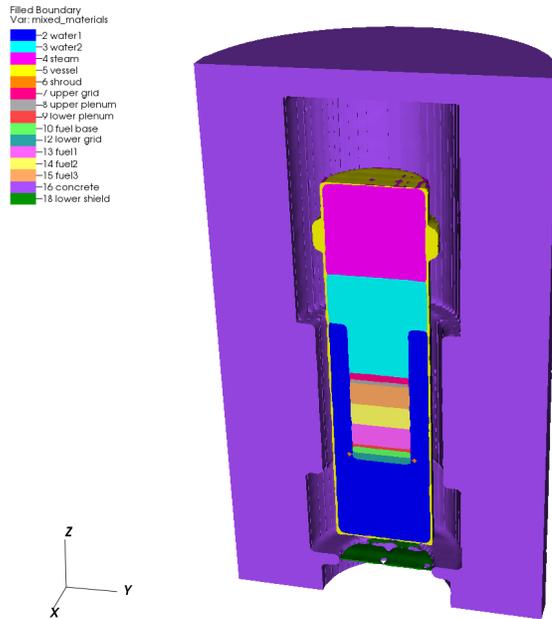


Fig. 7-24. Discretized material map for the JPDR problem.

To obtain effective source biasing parameters, the SDEF distribution bins defined in the original MCNP input file were partitioned into multiple bins. The original SDEF cards defined a cylindrical volume source with a Watt fission spectrum, as shown at the top of Fig. 7-25. Before running ADVANTG, SI and SP cards were added, as shown at the bottom of the figure. The radial and axial distributions were each subdivided into 10 equal-width bins. As in the Ueki shielding experiment problem, histogram bins were added for the Watt fission spectrum. Because of a limitation in MCNP5, only one continuous distribution can be biased at a time (see Section 9.4.1 for details). For this problem, the radial distribution was left as a continuous power law distribution. The Watt spectrum was converted from a continuous to a discrete distribution by replacing the original SP card with one containing bin-wise probabilities.

```

sdef pos=0 0 232.2 axs=0 0 1 rad=d1 ext=d2 erg=d3
sp1 -21 1
si1 0 64.79
si2 0 146.7
sp3 -3 0.965 2.29

```

original SDEF cards

```

sdef pos=0 0 232.2 axs=0 0 1 rad=d1 ext=d2 erg=d3
sp1 -21 1
si1 0 9i 64.79
c
si2 0 9i 146.7
sp2 0 1 9r
c
c sp3 -3 0.965 2.29 $ Removed for ADVANTG
c Watt spectrum (a = 0.96500, b = 2.29000)
c from 1.000e-11 to 20.0 MeV
c with 100 equiprobable bins below 6.000 MeV
c with 28 uniform bins above 6.000 MeV
si3 1.000000e-11 7.858811e-02 1.263175e-01 1.673049e-01 2.046618e-01
2.396678e-01 2.730056e-01 3.050934e-01 3.362108e-01 3.665564e-01
3.962780e-01 4.254896e-01 4.542815e-01 4.827270e-01 5.108870e-01
5.388127e-01 5.665480e-01 5.941312e-01 6.215957e-01 6.489714e-01
6.762850e-01 7.035610e-01 7.308216e-01 7.580875e-01 7.853778e-01
8.127106e-01 8.401029e-01 8.675711e-01 8.951307e-01 9.227968e-01
9.505839e-01 9.785065e-01 1.006579e+00 1.034814e+00 1.063226e+00
1.091829e+00 1.120637e+00 1.149663e+00 1.178920e+00 1.208425e+00
1.238189e+00 1.268229e+00 1.298559e+00 1.329194e+00 1.360151e+00
1.391445e+00 1.423093e+00 1.455113e+00 1.487522e+00 1.520340e+00
1.553587e+00 1.587282e+00 1.621448e+00 1.656106e+00 1.691282e+00
1.726999e+00 1.763285e+00 1.800167e+00 1.837675e+00 1.875841e+00
1.914698e+00 1.954281e+00 1.994630e+00 2.035786e+00 2.077791e+00
2.120693e+00 2.164543e+00 2.209396e+00 2.255312e+00 2.302354e+00
2.350593e+00 2.400106e+00 2.450975e+00 2.503294e+00 2.557164e+00
2.612695e+00 2.670013e+00 2.729255e+00 2.790575e+00 2.854147e+00
2.920166e+00 2.988854e+00 3.060464e+00 3.135286e+00 3.213656e+00
3.295965e+00 3.382672e+00 3.474322e+00 3.571566e+00 3.675194e+00
3.786177e+00 3.905727e+00 4.035382e+00 4.177138e+00 4.333654e+00
4.508577e+00 4.707110e+00 4.937053e+00 5.210882e+00 5.550488e+00
6.000000e+00 27i 2.000000e+01
sp3 0 9.756146e-03 99r 7.692682e-03 5.316894e-03 3.654824e-03
2.500049e-03 1.702589e-03 1.154847e-03 7.804437e-04 5.256420e-04
3.529249e-04 2.362746e-04 1.577544e-04 1.050638e-04 6.980737e-05
4.627946e-05 3.061754e-05 2.021614e-05 1.332351e-05 8.765429e-06
5.757063e-06 3.775179e-06 2.471812e-06 1.616088e-06 1.055150e-06
6.880009e-07 4.480366e-07 2.914139e-07 1.893216e-07 1.228578e-07

```

modified SDEF cards for ADVANTG

Fig. 7-25. User changes to SDEF cards for the JPDR problem.

ADVANTG was executed to generate variance reduction parameters using the FW-CADIS method. The forward and adjoint Denovo calculations took 71 and 74 CPU min, respectively. By executing the transport sweeps on multiple cores, the calculations took only about 36 min of wall-clock time. The other tasks performed by ADVANTG consumed only a few minutes.

The total forward and adjoint scalar fluxes from the Denovo calculations are shown in Fig. 7-26. From the center of the core to the far corner of the model, the forward flux varies over about 13 orders of magnitude. The default step characteristics differencing scheme was used, so is it reasonable to expect that the drop in the flux deep within the concrete is somewhat underestimated. Denovo also provides linear-discontinuous and trilinear-discontinuous differencing options, which are more accurate, but consume more memory and run time. For the purpose of generating variance reduction parameters, the step characteristics scheme has generally been found to be sufficient.

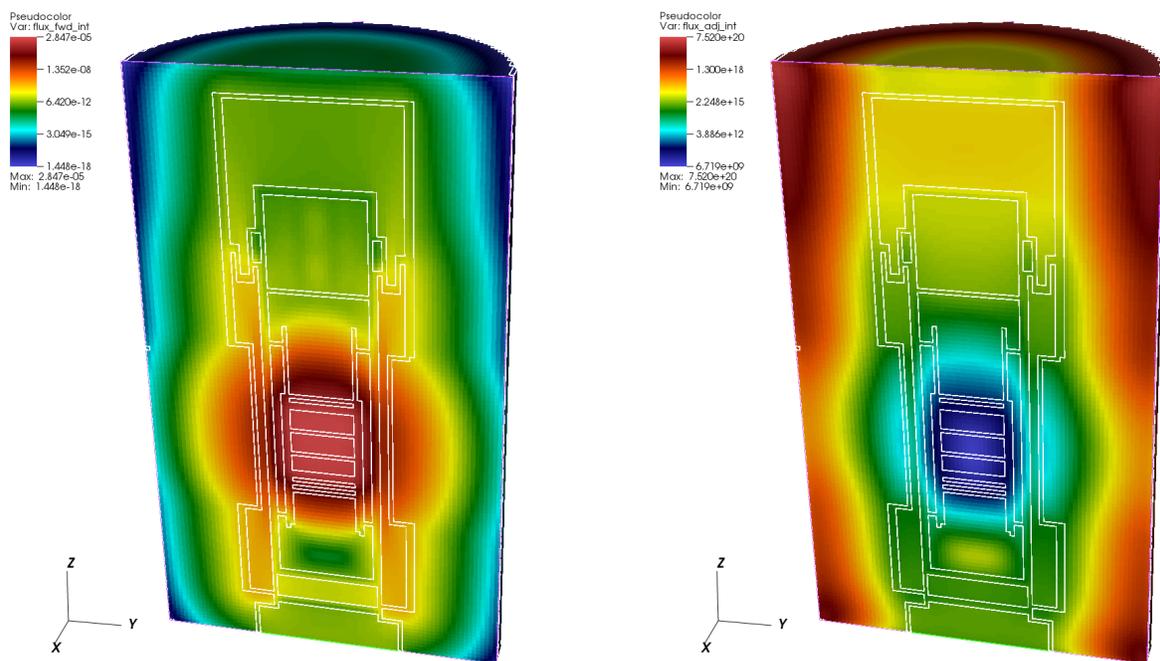


Fig. 7-26. Denovo forward (left) and adjoint (right) total flux for JPDR problem.

ADVANTG created a new MCNP input file and a WWINP file. The changes made to the MCNP input are summarized in Fig. 7-27. ADVANTG generated SB cards with importance-weighted biased probabilities for distributions 1, 2, and 3 and a WWP card. The radial biasing increases the probability of sampling particles in the two outermost radial bins, as shown in Table 7-9. The axial biasing mildly increases the probability of sampling particles near the top and bottom of the core, as shown in Table 7-10. Finally, the energy biasing parameters ensure that high-energy particles are sampled much more frequently than in an analog calculation. All of the biasing parameters encourage the sampling of particles that have a high probability of escaping the core.

```

sdef pos=0 0 232.2 axs=0 0 1 rad=d1 ext=d2 erg=d3
sp1 -21 1
si1 0 9i 64.79
c * added by ADVANTG
sb1 0.00000e+00 1.18068e-03 3.81923e-03 7.71686e-03 1.38558e-02
    2.38491e-02 4.24240e-02 7.66241e-02 1.38637e-01 2.44619e-01
    4.47274e-01
si2 0 9i 146.7
sp2 0 1 9r
c * added by ADVANTG
sb2 0.00000e+00 1.19687e-01 9.19858e-02 7.67686e-02 6.92604e-02
    6.85105e-02 7.27743e-02 8.31459e-02 1.02065e-01 1.32826e-01
    1.82977e-01
c sp3 -3 0.965 2.29 $ Removed for ADVANTG
c Watt spectrum (a = 0.96500, b = 2.29000)
si3 1.000000e-11 7.858811e-02 1.263175e-01 1.673049e-01 2.046618e-01
    ... omitted remainder of si3 card (next 20 lines)
sp3 0 9.756146e-03 99r 7.692682e-03 5.316894e-03 3.654824e-03
    ... omitted remainder of sp3 card (next 5 lines)
c * added by ADVANTG
sb3 0.00000e+00 7.92264e-05 9.98256e-05 1.09474e-04 1.19449e-04
    1.29539e-04 1.31757e-04 1.42252e-04 1.80775e-04 1.76305e-04
    1.77003e-04 1.74780e-04 1.78165e-04 1.73515e-04 1.84027e-04
    2.10024e-04 2.02329e-04 2.04453e-04 2.32284e-04 2.48785e-04
    2.57877e-04 2.51704e-04 2.49782e-04 2.75101e-04 2.99163e-04
    2.98298e-04 3.37556e-04 3.62211e-04 3.51235e-04 3.54748e-04
    3.57874e-04 3.58058e-04 3.78421e-04 5.26996e-04 5.39263e-04
    5.10395e-04 5.33903e-04 5.17355e-04 5.33640e-04 5.36325e-04
    5.34534e-04 5.37404e-04 5.25441e-04 5.32237e-04 6.46210e-04
    1.01776e-03 9.96315e-04 1.02320e-03 9.78025e-04 9.83560e-04
    1.04159e-03 9.76216e-04 1.02847e-03 1.06559e-03 1.71335e-03
    1.67775e-03 1.64756e-03 1.68783e-03 1.72450e-03 1.71231e-03
    1.73211e-03 2.74472e-03 3.04432e-03 3.05029e-03 3.07241e-03
    3.01304e-03 3.06688e-03 3.01714e-03 4.05700e-03 4.97481e-03
    5.13923e-03 6.92103e-03 7.19063e-03 8.33085e-03 9.07799e-03
    8.93173e-03 8.83296e-03 8.94436e-03 1.14671e-02 1.13922e-02
    1.17834e-02 1.14200e-02 1.30118e-02 1.35231e-02 1.35003e-02
    1.39870e-02 1.39522e-02 1.35874e-02 1.39041e-02 1.34226e-02
    2.72485e-02 2.78563e-02 2.74854e-02 2.75089e-02 2.70677e-02
    2.78565e-02 2.72168e-02 2.77040e-02 5.34031e-02 5.70908e-02
    5.76637e-02 8.45271e-02 6.18398e-02 4.46925e-02 3.81420e-02
    2.62320e-02 2.09088e-02 1.50367e-02 9.72226e-03 8.41462e-03
    5.75061e-03 3.73982e-03 2.51392e-03 2.06273e-03 1.58654e-03
    1.03821e-03 6.84569e-04 5.92152e-04 4.48678e-04 2.96386e-04
    1.92311e-04 1.25572e-04 8.08255e-05 5.39970e-05 3.58850e-05
    2.29718e-05 1.50102e-05 9.58401e-06 6.24761e-06
c * added by ADVANTG
wvp:n 5.0 j 100 j -1 0 4.161083969e+08

```

Fig. 7-27. ADVANTG changes to SDEF cards for the JPDR problem.

Table 7-9. ADVANTG radial biasing for the JPDR problem

Outer radius (cm)	Unbiased probability	Biased probability	Biased / unbiased
6.479	0.01	0.00118	0.118
12.958	0.03	0.00382	0.127
19.437	0.05	0.00772	0.154
25.916	0.07	0.0139	0.198
32.395	0.09	0.0238	0.265
38.874	0.11	0.0424	0.386
45.353	0.13	0.0766	0.589
51.832	0.15	0.139	0.924
58.311	0.17	0.245	1.439
64.79	0.19	0.447	2.354

Table 7-10. ADVANTG axial biasing for the JPDR problem

Center height (cm)	Unbiased probability	Biased probability	Biased / unbiased
7.335	0.1	0.120	1.197
22.005	0.1	0.0920	0.920
36.675	0.1	0.0768	0.768
51.345	0.1	0.0693	0.693
66.015	0.1	0.0685	0.685
80.685	0.1	0.0728	0.728
95.355	0.1	0.0831	0.831
110.025	0.1	0.102	1.021
124.695	0.1	0.133	1.328
139.365	0.1	0.183	1.830

7.3.5 Results

An MCNP simulation of the JPDR problem with the ADVANTG-generated variance reduction parameters was performed. As with the reference simulation, a run-time limit of 720 min was reached in an hour of wall-clock time using 12 threads. Plots of the total flux and associated relative uncertainties are shown in Fig. 7-28. The fraction of pulse-height bins with a relative uncertainty level at or below a given level is plotted in Fig. 7-29.

The FW-CADIS variance reduction parameters have a dramatic impact on the mesh tally results for this problem. Relatively uniform statistical uncertainties are obtained throughout the problem. Without variance reduction, only 1.7, 3, and 4% of the mesh tally voxels have relative errors less than or equal to 5, 10, and 15%. With the FW-CADIS variance reduction parameters, 36, 67, and 90% of the voxels have uncertainties at or below these levels. A conventional MCNP simulation would require an extremely long run time to obtain results converged at this level. With ADVANTG, the calculation of the variance reduction parameters and MCNP simulation could be performed on a desktop system in less than two hours of wall-clock time.

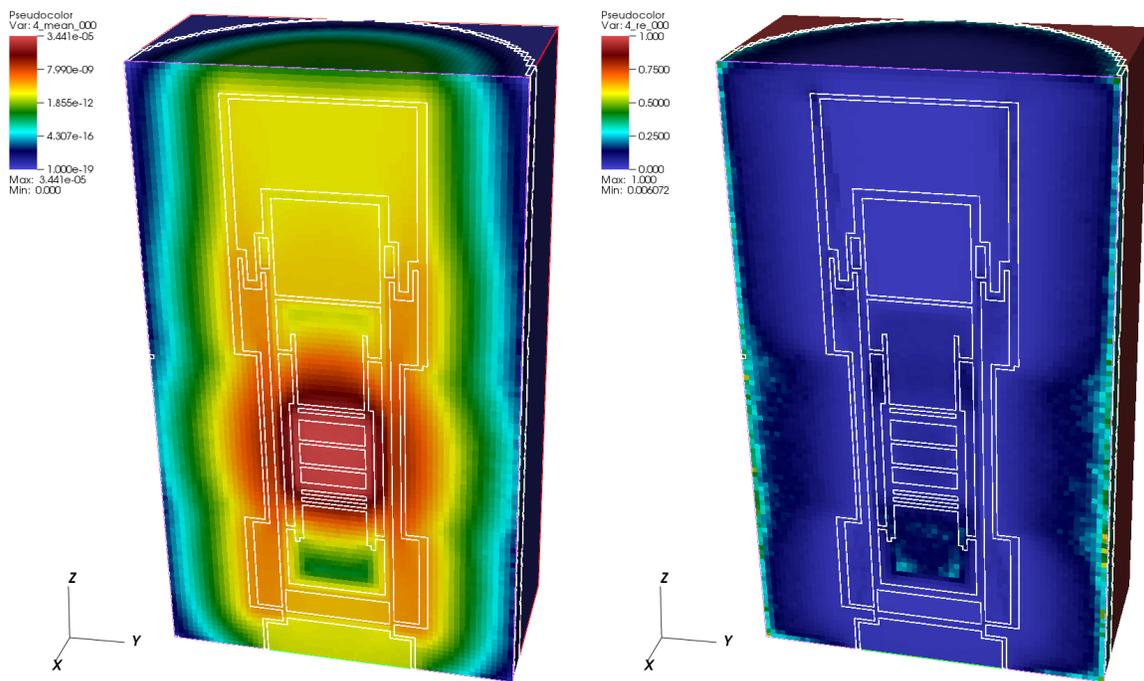


Fig. 7-28. ADVANTG-based total flux (left) and relative uncertainty (right) for the JPDR problem.

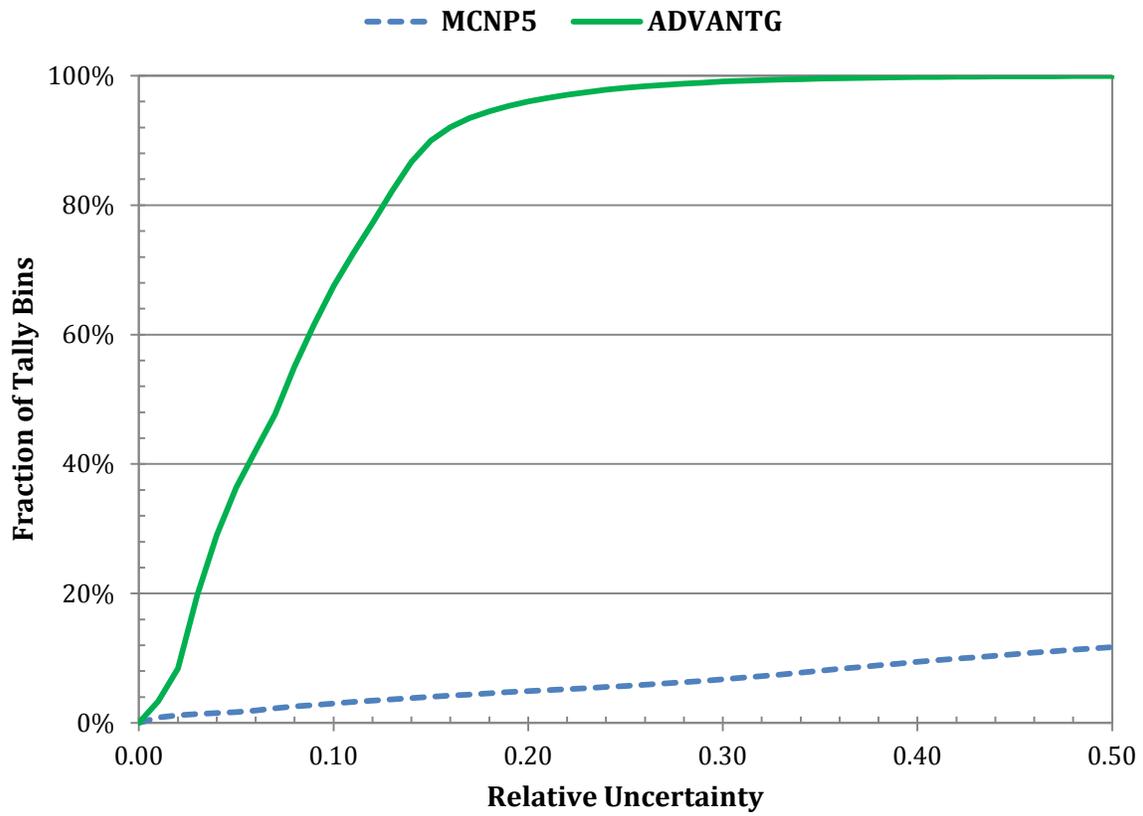


Fig. 7-29. Fraction of tally bins with less than a given uncertainty for the JPDR problem.

8. TIPS AND SUGGESTIONS

The authors have both applied and assisted with the application of ADVANTG in many challenging radiation shielding, radiation detection, neutron activation, and material damage/heating analyses and studies. This section provides suggestions, based on the authors' experience, for using ADVANTG to obtain accurate and precise tally estimates in MCNP simulations. The advice offered here is necessarily general in nature. However, the discussions may help to identify issues worth considering relative to a specific problem at hand.

8.1 SCOPING SIMULATIONS

Before applying ADVANTG to a problem, it is strongly recommended to perform an analog MCNP simulation with a time limit of at least 30 CPU minutes. Output from this initial run will provide insight into the characteristics of the transport problem and potentially indicate rates of tally convergence. In this regard, the problem summary table, the particle activity table, and the tally output tables all contain useful information. (Note that it may be necessary to add a PRINT card to the MCNP input file to obtain all relevant output.)

The problem summary table displays counts of various types of events in which particle tracks are created or destroyed. For example, the number of photon tracks created from neutron collisions, fluorescence, and bremsstrahlung are listed. Counts of tracks terminated due to escape, capture (for photons), and weight cutoff (for neutrons, due to implicit capture) are listed as well. The summary table also lists the average number of collisions per source particle. This quantity typically varies from a few to hundreds or even thousands of collisions per history, depending on type of particles being simulated and the composition and density of the material medium. Its value is highly correlated to the average cost of simulating a history.

The particle activity table (print table 126) lists the number of tracks entering each geometry cell, which can be used to gauge how frequently particles reach tally regions and their surroundings. The table also lists the total number of collisions sampled within each cell. From this information, the relative amount of simulation time spent tracking particles in the various parts of the geometry can be inferred.

The tally output tables, of course, provide a great deal of important information. At this initial stage, it is useful to examine the fraction of histories that contribute nonzero tally scores and the relative uncertainties of the tallies. In later stages, much more information should be reviewed; in particular, the ten statistical checks of tally convergence deserve careful study and consideration. At this point, if the relative uncertainty is less than about 20-25%, then the run time required to obtain a given level of uncertainty can generally be estimated with reasonable confidence.

Once a review of the scoping simulation is completed, an informed decision can be made as to whether or not it might be beneficial to apply ADVANTG to the problem. Certainly not all simulations require very long run times to obtain acceptably precise tally estimates (e.g., see the problem described in Section 7.1). If acceptably low relative uncertainties can be obtained in, say, a few to a half-dozen hours, and if most statistical checks are passed in the scoping run, then it is unlikely that there will be much to be gained by applying ADVANTG. In contrast, if very few tally scores are observed in the initial run, then it is worth considering whether or not applying weight windows and source biasing is likely to be effective at accelerating tally convergence.

The weight window technique consists of splitting and rouletting particle tracks based on space- and energy-dependent weight bounds. Splitting is helpful in problems, for example, where very few tracks reach regions of interest because of collisions in intervening materials. Similarly, rouletting is useful in problems where particles frequently collide in unimportant regions. On the other hand, neither splitting nor rouletting is particularly effective in problems where particles suffer only a few collisions on average and escape is the dominant mechanism by which particle tracks are terminated.

Source biasing is generally much more effective when used in combination with weight windows than when used by itself, though of course there are exceptions. Positional source biasing tends to be most effective in problems with large, distributed, optically-thick sources. Energy source biasing tends to be effective in problems where the tally regions are shielded from the source.

8.2 MCNP INPUT PREPARATION

ADVANTG currently requires MCNP input that is compatible with MCNP5-1.60. Inputs intended for MCNPX or MCNP6 may need to be modified before running ADVANTG. If this is the case, the `mcnp_input_template` option can be used to direct ADVANTG to merge generated SB and WWP cards into the original input file. See Section 5.8.1 for details.

It is recommended to review MCNP inputs that have been altered in the past to improve simulation performance. Not all modifications are neutral with regard to tally mean values. For example, truncating the geometry may eliminate regions that backscatter particles toward the tally or tallies of interest (e.g., room return or skyshine). Truncating the source, if not done carefully, can directly bias tally results. When effective weight windows and source biasing parameters are used, regions of the geometry and source that do not contribute strongly to tally scores will be sampled relatively less often than regions that contribute more strongly. Thus there is no need to cut off potentially important parts of the geometry or source.

When using ADVANTG, it is often necessary to partition source distributions in order to obtain effective biased probabilities (e.g., see the problem described in Section 7.3). Even continuous distributions (e.g., Watt fission spectra and power-law distributions) can be effectively subdivided by specifying histogram bins on an SI card. When these bins are present, ADVANTG will generate biased histogram probabilities for the continuous distribution. However, be careful to avoid biasing more than one continuous distribution at a time. See Sections 9.4.1 for details.

When using FW-CADIS path-length weighting to obtain relatively uniform Monte Carlo particle flux among multiple cell or surface tallies, it is important to be aware of the fact that ADVANTG treats each tally card (e.g., F4:n) as defining a single response region. This is true even if the tally card defines multiple cell or surface bins. If the cells or surfaces listed on a tally card differ significantly in volume or area, then the final relative uncertainties may be significantly nonuniform. To obtain relatively uniform uncertainties among multiple cells/surfaces it is necessary to define a tally for each cell/surface and to specify all of the tallies on the `mcnp_tallies` card.

There are implementation details to be aware of with regard to ADVANTG's treatment of tally multipliers. Though MCNP allows a tally to have more than one type of multiplier, ADVANTG will select and use only a single multiplier card to define the response spectrum. The order of precedence is DE/DF, FM, then EM. For example, if a tally has DE/DF cards, any FM or EM cards will be ignored. If no multiplier is defined, a uniform spectrum is used. It is also important to be aware that ADVANTG will use the first, and only the first, multiplier bin when constructing the tally response spectrum.

8.3 DISCRETE ORDINATES CALCULATIONS

In general, the most challenging aspect of using ADVANTG to generate effective variance reduction parameters is defining the spatial mesh, selecting the quadrature set, etc. for the discrete ordinates calculation(s). A typical starting point is a set of reasonable guesses informed by the physics of the problem and, where applicable, past experience. The default settings in ADVANTG, which cover everything except the multigroup library and spatial mesh, were selected based on the authors' experience with generating variance reduction parameters for a broad range of transport problems. Nonetheless, the defaults will not always provide solutions with sufficient fidelity.

With regard to defining the spatial mesh, it is recommended to use the `dx` option on the `method` card to generate visualization output before proceeding to execute the S_N solver. The mesh and material map can be inspected using `VisIt` to ensure that all important features are adequately represented in the discretized model. The edge of the mesh represents the boundary of the system in the discrete ordinates calculation. Hence it is important to include within the mesh boundaries all regions from which significant backscatter may occur.

It is generally advantageous to convert a very small volume source into a point source in order to activate a first-collision source calculation in Denovo. This treatment will help to mitigate ray effects and will produce a more accurate deterministic solution. For forward sources, the `mcnp_force_point_source` card can be used to force ADVANTG to treat any SDEF source as a point source. For adjoint sources (tallies), this must be done by manually defining a new F5 tally in the MCNP input.

Avoid placing mesh planes on or near point sources and point detectors. The first-collision source algorithm implemented in Denovo may produce nonphysical results if a forward or adjoint point source lies very close to a corner of a voxel. Ideally, point sources should be located at or near the center of a voxel.

Once a discrete ordinates solution has been generated, it is strongly recommended to visualize at least the energy-integrated fluxes using `VisIt`. The solution should be inspected for nonphysical features (e.g., ray effects and negativities) before executing any Monte Carlo simulations. If such features are present to a degree that they may impact the effectiveness of the variance reduction parameters, then the S_N solver settings should be refined and the calculation(s) should be re-executed. In addition, it is often useful to compare the structure of the spatial mesh to the adjoint flux in order to identify, and potentially refine, any large mesh intervals located in important regions of the problem.

ADVANTG can be used as a front end to drive Denovo calculations in situations where the discrete ordinates results are intended to be the final results. However, please note that the default settings for S_N calculations are intended for variance reduction parameter generation, where the accuracy requirements are generally not very high. For deterministic-only calculations, it is almost always necessary to increase the number of quadrature directions and Legendre scattering moments and to reduce convergence tolerances. Moreover, the broad-group cross section libraries distributed with ADVANTG may not provide sufficient fidelity for some applications. It is strongly recommended to compare the results of discrete ordinates calculations to continuous-energy Monte Carlo results for representative cases.

8.4 ADVANTG CALCULATIONS

The source biasing parameters generated by ADVANTG should be examined before proceeding to run an MCNP simulation. The difference between biased and unbiased probabilities should be proportional to the difference in expected tally contributions. If the shape of any of the biased probability distributions is

not compatible with an understanding of the transport physics of the problem, then new parameters should be generated before running MCNP. It may be necessary, for example, to refine the discrete ordinates spatial mesh in the vicinity of a source, or it may be necessary to select a more appropriate multigroup cross section library.

In some situations, it may be worthwhile to regenerate biased probabilities based on a different bin structure or an increased number of samples. Note that in these cases it is possible to recalculate biased source probabilities without re-running the discrete ordinates calculation(s). To do so, use the `-f` command line option when starting ADVANTG from the command line. See Section 4 for details.

In cases where spatial source biasing is used with cell or cookie-cutter rejection, it is extremely important to examine the relative uncertainty of the corrected SDEF WGT parameter, because this uncertainty directly impacts tally results. (See Section 3.8 for a discussion of issues related to using source biasing with cell rejection.) If the uncertainty is too large, it can be reduced by increasing the number of samples on the `mcnp_num_wgt_samples` card. As with source biasing parameters, the WGT parameter can be recalculated without re-running the discrete ordinates calculation(s).

8.5 MCNP SIMULATIONS

When using ADVANTG-generated weight windows and source biasing parameters, it is strongly recommended to perform at least one short MCNP simulation before proceeding to longer runs. There are two primary motivations for this. First, the average run time per history can increase significantly from an analog simulation. This is particularly true of FW-CADIS problems using global weighting with large tally regions. Second, tally convergence issues (as described in Section 8.6 below) can sometimes be identified at an early stage. When tally convergence is not robust, it is nearly always necessary to generate new variance reduction parameters, for example, by refining the discrete ordinates calculation(s), or perhaps by extending the mesh boundaries to include more of the problem domain. By checking for this type of problem early, the expense of a long and ultimately useless Monte Carlo run can be avoided.

It is not uncommon for ADVANTG-generated variance reduction parameters to cause an excessive amount of splitting in the MCNP simulation such that individual particle histories, either frequently or infrequently, take a very long time to complete. The presence of long histories can generally be confirmed by setting a simulation time limit, using the `CTME` card, and then observing that the simulation runs well beyond the limit. The MCNP `WWP` card provides a `MXSPLN` parameter that determines the maximum number of splits per event (i.e., surface crossing, collision, or mean free path of travel). However, reducing this limit is not generally effective at eliminating long histories.

In the authors' experience, there are two pathologies that most frequently cause long histories. The first is a discrepancy between the combinatorial and discretized geometries such that a streaming pathway or gap in the former is filled with significantly more dense material in the latter. In this case, the solution is to refine the spatial mesh in order to better resolve the feature and/or to trace a larger number of rays during the geometry mapping process to obtain more accurate mixed material fractions. The second pathology is an inaccurate adjoint solution in the low-energy groups where meshes tend to be large relative to the mean free path. The solution in this case is generally to omit one or more low-energy groups from the deterministic calculation(s) using the `denovo_last_group` card. In MCNP, the weight window parameters for the lowest given energy group extend to zero energy, so splitting and rouletting will still be applied to particles at these lower energies.

8.6 TALLY CONVERGENCE

When applying variance reduction (by any technique) or when using point detector tallies, it is extremely important to carefully review all tally output information. In particular, the ten statistical checks of tally convergence deserve careful study and consideration. See Volume I, Section 2.VI.I of the MCNP5 manual for details. Test failures serve as warnings that the tally confidence intervals may not be valid. In the authors' experience, failures of the variance-of-variance (VOV) and PDF slope tests are the strongest indicators of serious tally convergence problems in weight-window simulations.

It is extremely important to study the VOV trend printed in the tally fluctuation chart. The VOV is expected to decrease as $1/N$ after some early fluctuations, where N is the number of histories. If periodic increases in the VOV are observed throughout the simulation, or if the VOV remains at a high level, then this indicates that the tally scores include infrequent contributions from high-weight particles. This generally results from a deficiency in the discrete ordinates adjoint results, which should be examined to identify scoring pathways that are not accurately captured. This type of problem cannot be resolved by simply running more histories. New variance reduction parameters must be generated in order to obtain robust tally convergence.

Failure of the PDF slope test indicates that the high-score tail of the empirical tally score distribution is not well sampled. This may be due to a deficiency in the variance reduction parameters, or it may be due to an inadequate sample size (i.e., it may be resolved by simulating more particles). As with VOV test failures, if this test does not pass after running significantly more particles, one should look for scoring pathways that may not be accurately captured in the discrete ordinates calculation(s).

Mesh tally convergence is difficult to assess in a detailed way, because MCNP mesh tally output consists only of means and relative uncertainties. For this reason, it is strongly recommended to add one or more cell tallies within a mesh tally region in order to obtain more information about local tally convergence. If any convergence issues are observed with the cell tally, then the validity of the mesh tally results must be treated with skepticism until the issues can be resolved.

Point detectors can have interesting pathologies (see Volume I, Sections 2.V.E and 2.VI.J of the MCNP5 manual for details). Because point detector contributions are scored from every source and scattering event, point detector tallies rarely suffer from a lack of scores. However, the spatial distribution of the sampled scattering events must be considered when assessing the accuracy of point detector results. ADVANTG-generated variance reduction parameters will generally change the distribution of sampled scatters, sometimes with a surprisingly negative impact on apparent point detector convergence. This is often due to the sampling of more scattering events near the point detector, which can lead to a higher and more accurate estimate of the variance relative to an analog simulation. In cases where point detector convergence seems elusive, ensure that the point is not placed within or near a scattering material. Even when this condition is met, it may be necessary to move the point detector, or to replace it with a small, but not too small, cell tally.

9. LIMITATIONS

The implementations of the CADIS and FW-CADIS methods in ADVANTG are based on the use of scalar adjoint flux estimates from Denovo calculations. As a result, no directional biasing, in either the weight-window parameters or the biased source distributions, is currently implemented.

Support for various MCNP SDEF source and tally features is discussed in Sections 9.1 and 9.2, respectively. Limitations in Denovo and MCNP5 are described in Sections 9.3 and 9.4.

9.1 SUPPORTED MCNP SDEF DISTRIBUTION TYPES

ADVANTG samples from SDEF source distributions on two occasions during FW-CADIS calculations: once to map the source onto the deterministic mesh and once to estimate source biasing parameters. In CADIS calculations, only the latter is required. ADVANTG does not currently support SDEF sources that use transformations. In addition, sources that have circular dependencies within distribution trees are not supported.

The use of DS H, DS L, and DS T cards for the ERG variable is not currently supported for source mapping, because this would require partitioning the energy distribution bins in a spatially dependent manner. For the same reason, if ERG is an independent variable that is depended upon by another variable, it must be specified as an S distribution.

The algorithm implemented in ADVANTG for estimating a corrected SDEF WGT parameter when spatial source biasing is used with cell or cookie-cutter rejection does not support:

- Built-in function biasing of the RAD or EXT variables using SB -21 or -31 cards
- Rejection over multiple geometry or cookie-cutter cells

An error message will be printed in either of these cases. It is possible to disable the generation of spatial source biasing parameters using the `mcnp_sb_type` card. However, this may cause a significant fraction of source particles to be created with weights outside of the weight-window bounds, which can negatively impact tally convergence. Use this workaround with caution.

9.2 SUPPORTED MCNP TALLIES

ADVANTG maps tally spatial regions and response spectra onto the deterministic space-energy mesh for use as the adjoint source. The tally types that are and are not supported by ADVANTG are listed in Table 9-1. Note that ADVANTG supports rectangular mesh tallies (FMESHn card with GEOM=xyz keyword option), but it does not currently support cylindrical mesh tallies (GEOM=cy1) or mesh tally transformations.

ADVANTG supports the use of FM card tally multipliers with multiplier sets that include either the special negative reaction numbers listed in Table 3.5 in MCNP Manual, Vol. II, Sec. 3.IV.E.7, or arbitrary MT reaction numbers. ADVANTG does not support FM card attenuator sets or the zero material number option for mesh tallies.

Because pulse-height estimation is not generally available in deterministic solvers, F8 tallies are treated as track-length cell tallies that have a total cross section multiplier. This treatment seems to be sufficient for most problems.

Table 9-1. Supported MCNP5 tally types and tally modifiers

Supported	Not supported
F1	*F1, *F2, *F4, *F5
F2	F5X, F5Y, F5Z
F4	FIP5, FIR5, FIC5
F5	F7
F6	+F8
F8	FMESHn GEOM=cyl
FMESHn GEOM=xyz	
En	DXT
FMn	
DE/DFn	
EMn	

When a tally must be modified solely for the purpose of running ADVANTG, the `mcnp_input_template` card can be used to direct ADVANTG to merge the generated WWP and SB cards into the original MCNP input deck (i.e., the deck that contains the original tally cards).

9.3 LIMITATIONS IN DENOVO

In Denovo, fission is treated as capture when performing fixed-source calculations. As a result, the variance reduction parameters generated by ADVANTG do not account for particles emitted by induced fission events. This limitation is particularly relevant to certain types of active interrogation and nondestructive assay problems. The recommended method to apply ADVANTG to these types of problems is to break the calculation into two steps: first estimate the distribution of induced fissions, and then transport the particles emitted from fission to the detector. This process is labor intensive, because it requires converting a fission neutron production tally into an SDEF source specification for MCNP. This approach also introduces some discretization error as a result of tallying the fission source in the first step.

Denovo does not treat photonuclear reactions. None of the multigroup libraries that are distributed with ADVANTG contain photonuclear data.

9.4 LIMITATIONS IN MCNP5

9.4.1 Biasing Continuous Distributions

MCNP5 supports the biasing of a continuous distribution (e.g., Watt fission spectrum) with a histogram (SI H) or linearly interpolated (SI A) distribution. In this case, MCNP replaces the continuous distribution with an equiprobable bin representation that uses at most 300 bins (X-5 Monte Carlo Team 2003, Sec. 2.VII.B.10.c). MCNP constructs an equal number of equiprobable bins in each histogram interval. This implementation introduces a bias that can be significant when the histogram contains one or more broad intervals. (MCNP prints a warning message in this case.) For this reason, caution should be exercised when applying source biasing parameters to continuous distributions. Using fine-grained histogram intervals is recommended for the biased distribution (e.g., using the `watt.py` script from the `share/` directory of the ADVANTG installation).

MCNP5 versions up to and including 1.60 do not correctly store equiprobable bin boundaries for more than one distribution. This issue causes source particles to be sampled from an incorrect distribution and can usually be observed by studying the starting coordinates of the first 50 source particles (MCNP print table 110). Because of this behavior, only one continuous source distribution should be biased at a time. For problems where it is desired to bias more than one continuous distribution, converting the remaining distributions to histogram or interpolated distributions is recommended. Caution should be exercised to ensure that this substitution does not introduce a significant bias into the final results.

9.4.2 Pulse-Height Tallies

Recent versions of MCNP5 support the use of the weight-window variance reduction technique in problems with pulse-height (F8) tallies. Pulse-height tally variance reduction requires extra bookkeeping to produce correct tally estimates in the presence of particle splitting and consumes additional memory. In most problems, the extra memory usage is not noticeable. However, in problems where there are large numbers of splitting events per history, the memory usage can be quite high. For example, in a simulation where a pulse-height tally was placed 50 m from the source, MCNP5 consumed more than 16 GB of memory when ADVANTG-generated weight windows were used. The memory usage dropped to less than 100 MB when the F8 tally was replaced with an F4 tally. For this reason, monitoring the memory consumed by MCNP5 when using the weight-window technique with pulse-height tallies is recommended.

MCNP does not allow weight windows to be used with pulse-height tallies in (n, γ) problems. Only source biasing parameters can be used in these cases.

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APPENDIX A. MULTIGROUP ENERGY BOUNDS

Table A-1. 27n19g library groups

Neutron groups		Photon groups	
Group	Upper energy boundary (MeV)	Group	Upper energy boundary (MeV)
0	2.0000E+01	27	2.0000E+01
1	6.3763E+00	28	1.0000E+01
2	3.0119E+00	29	8.0000E+00
3	1.8268E+00	30	6.5000E+00
4	1.4227E+00	31	5.0000E+00
5	9.0718E-01	32	4.0000E+00
6	4.0762E-01	33	3.0000E+00
7	1.1109E-01	34	2.5000E+00
8	1.5034E-02	35	2.0000E+00
9	3.0354E-03	36	1.6600E+00
10	5.8295E-04	37	1.3300E+00
11	1.0130E-04	38	1.0000E+00
12	2.9023E-05	39	8.0000E-01
13	1.0677E-05	40	6.0000E-01
14	3.0590E-06	41	4.0000E-01
15	1.8554E-06	42	3.0000E-01
16	1.3000E-06	43	2.0000E-01
17	1.1253E-06	44	1.0000E-01
18	1.0000E-06	45	4.5000E-02
19	8.0000E-07		1.0000E-02
20	4.1399E-07		
21	3.2500E-07		
22	2.2500E-07		
23	1.0000E-07		
24	5.0000E-08		
25	3.0000E-08		
26	1.0000E-08		
	1.0000E-11		

Table A-2. 200n47g library groups

Neutron groups				Photon groups	
Group	Upper energy boundary (MeV)	Group	Upper energy boundary (MeV)	Group	Upper energy boundary (MeV)
0	2.0000E+01	50	2.0190E+00	200	2.0000E+01
1	1.9640E+01	51	1.9205E+00	201	1.4000E+01
2	1.7332E+01	52	1.8268E+00	202	1.2000E+01
3	1.6905E+01	53	1.7377E+00	203	1.0000E+01
4	1.6487E+01	54	1.6530E+00	204	8.0000E+00
5	1.5683E+01	55	1.5724E+00	205	7.5000E+00
6	1.4918E+01	56	1.4957E+00	206	7.0000E+00
7	1.4550E+01	57	1.4227E+00	207	6.5000E+00
8	1.4191E+01	58	1.3534E+00	208	6.0000E+00
9	1.3840E+01	59	1.2874E+00	209	5.5000E+00
10	1.3499E+01	60	1.2246E+00	210	5.0000E+00
11	1.2840E+01	61	1.1648E+00	211	4.5000E+00
12	1.2523E+01	62	1.1080E+00	212	4.0000E+00
13	1.2214E+01	63	1.0026E+00	213	3.5000E+00
14	1.1618E+01	64	9.6164E-01	214	3.0000E+00
15	1.1052E+01	65	9.0718E-01	215	2.7500E+00
16	1.0513E+01	66	8.6294E-01	216	2.5000E+00
17	1.0000E+01	67	8.2085E-01	217	2.3500E+00
18	9.5123E+00	68	7.8082E-01	218	2.1500E+00
19	9.0484E+00	69	7.4274E-01	219	2.0000E+00
20	8.6071E+00	70	7.0651E-01	220	1.8000E+00
21	8.1873E+00	71	6.7206E-01	221	1.6600E+00
22	7.7880E+00	72	6.3928E-01	222	1.5700E+00
23	7.4082E+00	73	6.0810E-01	223	1.5000E+00
24	7.0469E+00	74	5.7844E-01	224	1.4400E+00
25	6.7032E+00	75	5.5023E-01	225	1.3300E+00
26	6.5924E+00	76	5.2340E-01	226	1.2000E+00
27	6.3763E+00	77	4.9787E-01	227	1.0000E+00
28	6.0653E+00	78	4.5049E-01	228	9.0000E-01
29	5.7695E+00	79	4.0762E-01	229	8.0000E-01
30	5.4881E+00	80	3.8774E-01	230	7.0000E-01
31	5.2205E+00	81	3.6883E-01	231	6.0000E-01
32	4.9659E+00	82	3.3373E-01	232	5.1200E-01
33	4.7237E+00	83	3.0197E-01	233	5.1000E-01
34	4.4933E+00	84	2.9849E-01	234	4.5000E-01
35	4.0657E+00	85	2.9721E-01	235	4.0000E-01
36	3.6788E+00	86	2.9452E-01	236	3.0000E-01
37	3.3287E+00	87	2.8725E-01	237	2.6000E-01
38	3.1664E+00	88	2.7324E-01	238	2.0000E-01
39	3.0119E+00	89	2.4724E-01	239	1.5000E-01
40	2.8651E+00	90	2.3518E-01	240	1.0000E-01
41	2.7253E+00	91	2.2371E-01	241	7.5000E-02
42	2.5924E+00	92	2.1280E-01	242	7.0000E-02
43	2.4660E+00	93	2.0242E-01	243	6.0000E-02
44	2.3852E+00	94	1.9255E-01	244	4.5000E-02
45	2.3653E+00	95	1.8316E-01	245	3.0000E-02
46	2.3457E+00	96	1.7422E-01	246	2.0000E-02
47	2.3069E+00	97	1.6573E-01		1.0000E-02
48	2.2313E+00	98	1.5764E-01		
49	2.1225E+00	99	1.4996E-01		

continued on next page

Table A-2 (cont'd)

Neutron groups			
Group	Upper energy boundary (MeV)	Group	Upper energy boundary (MeV)
100	1.4264E-01	150	1.6702E-04
101	1.3569E-01	151	1.3007E-04
102	1.2907E-01	152	1.0130E-04
103	1.2277E-01	153	7.8893E-05
104	1.1679E-01	154	6.1442E-05
105	1.1109E-01	155	4.7851E-05
106	9.8037E-02	156	3.7266E-05
107	8.6517E-02	157	2.9023E-05
108	8.2503E-02	158	2.2603E-05
109	7.9499E-02	159	1.7604E-05
110	7.1998E-02	160	1.3710E-05
111	6.7379E-02	161	1.0677E-05
112	5.6562E-02	162	8.3153E-06
113	5.2475E-02	163	6.4760E-06
114	4.6309E-02	164	5.0435E-06
115	4.0868E-02	165	3.9279E-06
116	3.4307E-02	166	3.0590E-06
117	3.1828E-02	167	2.3824E-06
118	2.8501E-02	168	1.8554E-06
119	2.7000E-02	169	1.4450E-06
120	2.6058E-02	170	1.3000E-06
121	2.4788E-02	171	1.1253E-06
122	2.4176E-02	172	1.0800E-06
123	2.3579E-02	173	1.0400E-06
124	2.1875E-02	174	1.0000E-06
125	1.9305E-02	175	8.7643E-07
126	1.5034E-02	176	8.0000E-07
127	1.1709E-02	177	6.8256E-07
128	1.0595E-02	178	6.2506E-07
129	9.1188E-03	179	5.3158E-07
130	7.1017E-03	180	5.0000E-07
131	5.5308E-03	181	4.1399E-07
132	4.3074E-03	182	3.6680E-07
133	3.7074E-03	183	3.2500E-07
134	3.3546E-03	184	2.7500E-07
135	3.0354E-03	185	2.2500E-07
136	2.7465E-03	186	1.8400E-07
137	2.6126E-03	187	1.5000E-07
138	2.4852E-03	188	1.2500E-07
139	2.2487E-03	189	1.0000E-07
140	2.0347E-03	190	7.0000E-08
141	1.5846E-03	191	5.0000E-08
142	1.2341E-03	192	4.0000E-08
143	9.6112E-04	193	3.0000E-08
144	7.4852E-04	194	2.1000E-08
145	5.8295E-04	195	1.4500E-08
146	4.5400E-04	196	1.0000E-08
147	3.5357E-04	197	5.0000E-09
148	2.7536E-04	198	2.0000E-09
149	2.1445E-04	199	5.0000E-10
			1.0000E-11

Table A-3. BUGLE-96 and BPLUS library groups

Neutron groups		Photon groups	
Group	Upper energy boundary (MeV)	Group	Upper energy boundary (MeV)
0	1.7332E+01	47	1.4000E+01
1	1.4191E+01	48	1.0000E+01
2	1.2214E+01	49	8.0000E+00
3	1.0000E+01	50	7.0000E+00
4	8.6071E+00	51	6.0000E+00
5	7.4082E+00	52	5.0000E+00
6	6.0653E+00	53	4.0000E+00
7	4.9659E+00	54	3.0000E+00
8	3.6788E+00	55	2.0000E+00
9	3.0119E+00	56	1.5000E+00
10	2.7253E+00	57	1.0000E+00
11	2.4660E+00	58	8.0000E-01
12	2.3653E+00	59	7.0000E-01
13	2.3457E+00	60	6.0000E-01
14	2.2313E+00	61	4.0000E-01
15	1.9205E+00	62	2.0000E-01
16	1.6530E+00	63	1.0000E-01
17	1.3534E+00	64	6.0000E-02
18	1.0026E+00	65	3.0000E-02
19	8.2085E-01	66	2.0000E-02
20	7.4274E-01		1.0000E-02
21	6.0810E-01		
22	4.9787E-01		
23	3.6883E-01		
24	2.9721E-01		
25	1.8316E-01		
26	1.1109E-01		
27	6.7379E-02		
28	4.0868E-02		
29	3.1828E-02		
30	2.6058E-02		
31	2.4176E-02		
32	2.1875E-02		
33	1.5034E-02		
34	7.1017E-03		
35	3.3546E-03		
36	1.5846E-03		
37	4.5400E-04		
38	2.1445E-04		
39	1.0130E-04		
40	3.7266E-05		
41	1.0677E-05		
42	5.0435E-06		
43	1.8554E-06		
44	8.7643E-07		
45	4.1399E-07		
46	1.0000E-07		
	1.0000E-11		

Table A-4. DABL69 and DPLUS library groups

Neutron groups		Photon groups	
Group	Upper energy boundary (MeV)	Group	Upper energy boundary (MeV)
0	1.9640E+01	46	2.0000E+01
1	1.6905E+01	47	1.4000E+01
2	1.4918E+01	48	1.2000E+01
3	1.4191E+01	49	1.0000E+01
4	1.3840E+01	50	8.0000E+00
5	1.2523E+01	51	7.0000E+00
6	1.2214E+01	52	6.0000E+00
7	1.1052E+01	53	5.0000E+00
8	1.0000E+01	54	4.0000E+00
9	9.0484E+00	55	3.0000E+00
10	8.1873E+00	56	2.5000E+00
11	7.4082E+00	57	2.0000E+00
12	6.3763E+00	58	1.5000E+00
13	4.9659E+00	59	1.0000E+00
14	4.7237E+00	60	7.0000E-01
15	4.0657E+00	61	4.5000E-01
16	3.0119E+00	62	3.0000E-01
17	2.3852E+00	63	1.5000E-01
18	2.3069E+00	64	1.0000E-01
19	1.8268E+00	65	7.0000E-02
20	1.4227E+00	66	4.5000E-02
21	1.1080E+00	67	3.0000E-02
22	9.6164E-01	68	2.0000E-02
23	8.2085E-01		1.0000E-02
24	7.4274E-01		
25	6.3928E-01		
26	5.5023E-01		
27	3.6883E-01		
28	2.4724E-01		
29	1.5764E-01		
30	1.1109E-01		
31	5.2475E-02		
32	3.4307E-02		
33	2.4788E-02		
34	2.1875E-02		
35	1.0333E-02		
36	3.3546E-03		
37	1.2341E-03		
38	5.8295E-04		
39	2.7536E-04		
40	1.0130E-04		
41	2.9023E-05		
42	1.0677E-05		
43	3.0590E-06		
44	1.1253E-06		
45	4.1399E-07		
	1.0000E-11		

Table A-5. FENDL67 library groups

Neutron groups		Photon groups	
Group	Upper energy boundary (MeV)	Group	Upper energy boundary (MeV)
0	1.4191e+01	46	1.4000e+01
1	1.3499e+01	47	1.2000e+01
2	1.2214e+01	48	1.0000e+01
3	1.1052e+01	49	8.0000e+00
4	1.0000e+01	50	7.5000e+00
5	9.0484e+00	51	7.0000e+00
6	8.1873e+00	52	6.5000e+00
7	7.4082e+00	53	6.0000e+00
8	6.7032e+00	54	5.5000e+00
9	6.0653e+00	55	5.0000e+00
10	5.4881e+00	56	4.5000e+00
11	4.9659e+00	57	4.0000e+00
12	4.4933e+00	58	3.5000e+00
13	4.0657e+00	59	3.0000e+00
14	3.6788e+00	60	2.5000e+00
15	3.3287e+00	61	2.0000e+00
16	3.0119e+00	62	1.5000e+00
17	2.7253e+00	63	1.0000e+00
18	2.4660e+00	64	4.0000e-01
19	1.8268e+00	65	2.0000e-01
20	1.3534e+00	66	1.0000e-01
21	1.0026e+00		1.0000e-02
22	7.4274e-01		
23	5.5023e-01		
24	4.0762e-01		
25	3.0197e-01		
26	2.2371e-01		
27	1.6573e-01		
28	1.2277e-01		
29	6.7379e-02		
30	3.1828e-02		
31	1.5034e-02		
32	7.1017e-03		
33	3.3546e-03		
34	1.5846e-03		
35	7.4852e-04		
36	3.5358e-04		
37	1.6702e-04		
38	7.8893e-05		
39	3.7267e-05		
40	1.7603e-05		
41	8.3153e-06		
42	3.9279e-06		
43	1.8554e-06		
44	8.7642e-07		
45	4.1399e-07		
	1.0000e-11		

APPENDIX B. INDEX OF FREQUENTLY USED KEYWORDS

The following table lists frequently used keywords. Keywords might be required in general (R) or only in the given context (C).

Keyword	Brief description	Req'd	Page
model	Type of transport problem model		24
method	Variance reduction method or type of computational sequence	R	24
anisn_library	Name of the ANISN-format cross section library	R	32
mesh_x , etc.	Coordinates of mesh voxel edges in the x , y , and z dimensions, in cm	R	30
denovo_x_blocks , etc.	Number of spatial domain partitions in the x and y dimensions, and the number of pipelining blocks in the z dimension		34
denovo_quadrature	Type of quadrature set. See also: denovo_quad_num_azi, denovo_quad_num_polar, denovo_quad_order, denovo_quad_polar_axis		34
denovo_pn_order	Order of Legendre scattering-angle expansion		37
denovo_first_group , denovo_last_group	The first and last energy groups to be solved		39
Context: model mcnp			
mcnp_input	Filename of the MCNP5 input file	C	25
mcnp_tallies	MCNP tally numbers for which to generate variance reduction parameters and/or to use in forming adjoint sources.	C	25
mcnp_target_sb_density	Target average number of samples per SDEF distribution bin when generating biased source probabilities.		70
Context: model sword			
sword_input	Filename of the .sword file	C	27
Context: method fwcadis			
fwcadis_spatial_treatment	Spatial treatment used in forming the FW-CADIS adjoint source.		29
fwcadis_response_weighting	If True, form the adjoint source to accelerate an energy-integrated response (e.g., dose or total flux)		29
Context: method dx			
dx_forward , dx_adjoint	If True, executes a deterministic calculation in the specified transport mode		30