

# **Core Average (MOX/LEU) Nuclide Ratios for Sequoyah and Browns Ferry Reactors (Nonproprietary)**

**April 2013**

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Reactor and Nuclear Systems Division

**CORE AVERAGE (MOX/LEU) NUCLIDE RATIOS FOR  
SEQUOYAH AND BROWNS FERRY REACTORS (NONPROPRIETARY)  
PROBLEM DESCRIPTION, INPUTS, AND RESULTS**

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## 1. INTRODUCTION

Oak Ridge National Laboratory (ORNL) was directed by the Department of Energy NN-26 to develop and run SCALE/TRITON [1] models for the generation of core-average nuclide inventories, in a MOX core (61% LEU + 39% MOX) and a LEU core with assembly burnups as high as ~60 GWd/MTU, for a Sequoyah pressurized water reactor (PWR) and a Browns Ferry boiling water reactor (BWR). This study compares the nuclide ratios of a representative MOX core to a representative LEU core for both the Sequoyah (SQN) and Browns Ferry (BFN) reactors. Previous work [2, 3], supporting earlier Environmental Impact Statement (EIS) reports, considered only a PWR assembly.

The previous work [2, 3] determined the nuclide ratios assuming a LEU assembly with a uniform loading of 4.37 wt% U-235 enrichment fuel rods and a MOX assembly with a uniform loading of 4.37 wt% (Pu-Am) assay fuel rods. The intention in the current study is to utilize the mechanical design of the actual SQN and BFN assemblies with current proposed core enrichment loadings (LEU and MOX) at SQN and BFN. This deviates from the philosophy of the original studies that compared identical but hypothetical enrichment loadings and is deemed to be more appropriate in light of the current status of program development.

In the current work, the same weapons-grade (Pu-Am) vector is used in the MOX fuel for both SQN and BFN. Since the amount of (Pu-Am) in a fuel rod is not, strictly speaking, an enrichment, it is referred to in this report as an “assay.”

The W17 HTP assembly configuration [4] is used in this work for the modeling of the SQN reactors. The assembly characteristics are close to but not identical to the values used for the previous calculations [2].

The SQN data reflects three different (Pu-Am) assays in the W17 HTP MOX assembly. The nominal average (Pu-Am) assay for this assembly is 4.35 wt% in the current study compared to the 4.37 wt% assay of the earlier study.

For the BFN reactors, the analysis uses the ATRIUM 10×10 fuel assembly design. This assembly is quite heterogeneous with respect to the number of (LEU+Gd) rods, the locations of these rods in the assembly, or the Gd<sub>2</sub>O<sub>3</sub> content in (LEU+Gd) fuel used in these rods. A MOX assembly using only four (Pu-Am) assays was selected as the candidate assembly design for the BFN reactors.

## 2. BACKGROUND

The original work [2] was developed in 1995 using simple models and computational capabilities as available at that time. Since then, there have been substantial improvements in code capabilities and nuclear data. The computational analysis in the current report has been performed using only the SCALE nuclear analysis code system [1]. The current modeling and simulation capabilities in SCALE allow a full two-dimensional (2D) representation of the fuel assembly, an improved cross section self-shielding treatment, and the use of individual fuel rod Dancoff factors, whereas the models used in Reference [2] used a simplified one-dimensional (1D) representation of the fuel assembly and older versions of nuclear data. The current SCALE computational capabilities include an improved 238-group ENDF-B/VII cross section library that is based on the most recent cross section data evaluations, whereas a 27-group cross section library was used in Reference [2], which was based mostly on ENDF-B/IV cross sections with some ENDF-B/V cross sections data included.

### 3. PROBLEM DEFINITION AND METHOD OF ANALYSIS

This section presents a description of the problem and an overview of the analysis process.

To obtain the nuclide ratios required for the EIS report, it was necessary to determine the average concentration of all required nuclides in each of the MOX and LEU equilibrium cores for each reactor type (SQN, BFN). The number and type of assemblies and the distribution of assembly burnups in the equilibrium cores were determined by fuel management studies [5, 6, 7 and 8].

#### 3.1 Sequoyah Cores

The SQN equilibrium MOX core consists of 193 fuel assemblies that are a mix of LEU and MOX fuel assemblies. The weighted average burnup for the SQN MOX core is 37.10 GWd/MTHM.

The SQN equilibrium LEU core consists of 100% LEU fuel assemblies with three enrichments. The weighted average burnup for the SQN LEU core is 38.06 GWd/MTHM.

#### 3.2 Browns Ferry Cores

The BFN equilibrium MOX core consists of 764 fuel assemblies that are a mix of LEU and MOX fuel assemblies. The average burnup for the BFN MOX core is 33.08 GWd/MTHM.

The BFN equilibrium LEU core consists of 100% LEU fuel assemblies. The weighted average burnup for the BFN LEU core is 35.46 GWd/MTHM.

#### 3.3 Core Averaging of Nuclide Loads and Nuclide Ratios

The core (Pu-Am) assay/U-235 enrichment and burnup loading maps were obtained from the fuel management studies that defined the equilibrium MOX and LEU cores for each reactor type. Using these maps, it is possible to determine the number of each type of assembly in each type of core. As noted, each type of assembly can be characterized into two or three burnup classes (low, medium, high). The average burnup for each class was determined for each assembly type. Thus, it could be determined what fraction of the core loading is represented by each burnup class of each assembly. Depletion simulations were performed for each assembly type model to deplete to ~60 GWd/MTHM, thus covering the entire range of assembly burnups in the core. These data served to interpolate for calculating the nuclide concentrations that correspond to the average burnups of the burnup classes for each assembly type.

The nuclide concentrations thus determined were summed and weighted by the core fraction of that burnup class for each assembly type to produce a core-average value of the nuclide concentrations. The nuclide concentration ratios were determined from these weighted sums for the MOX and LEU cores. The core loads were determined by multiplying the concentrations (g/MTHM) by the MTHM per assembly and then by the number of assemblies in the core.

#### 3.4 Assembly Modeling

Previous studies [9, 10] have highlighted the need to calculate fuel rod Dancoff factors independently of the standard TRITON/MIPLIB process for highly heterogeneous BWR fuel assemblies. The SCALE module MCDANCOFF was employed to calculate Dancoff factors for every fuel rod in the assembly. Dancoff factor maps were therefore generated for the ATRIUM 10 dominant and vanished lattices (short rods replaced by moderator) at void concentrations of 0%, 40%, and 80%. Using a typical void profile, an

average void of 25% was determined for the dominant lattice and an average void of 70% was determined for the vanished lattice. The void profiles are almost identical for the BFN MOX and LEU assemblies. Hence, only the two void levels described above were used. These void concentrations were used in the depletion calculations of each type of lattice. This work was carried out using SCALE version 6.1, which permits the use of MCDANCOFF-calculated Dancoff factors.

This project used a format for TRITON input files that is based on naming fuels, used in the models, by their array positions, thus making the descriptions clear and explicit. This strategy leads to relatively large input files (>1000 lines) but promotes consistency and clear understanding of the model.

#### **4. DESIGN DOCUMENTS**

References [4 to 8] are the base documents used to develop the MOX and LEU models for this study. Reference [4] was not used directly. However, by means of a private communication with AREVA, the physical details of the W17 HTP assembly were confirmed. Reference [5] reports the results of a fuel cycle study to produce an equilibrium MOX core in the Sequoyah reactors; the assembly assay and enrichment loadings and end of cycle (EOC) burnup distribution were taken from this reference. Reference [6] reports a fuel cycle study of an all LEU core using commercial-grade uranium (CGU) in the Sequoyah reactors; the assembly enrichments and EOC burnup distribution for the SQN equilibrium LEU core were taken from this reference.

Reference [7] describes a study of both an equilibrium LEU core and two variations of a MOX core in a Browns Ferry reactor. The information for the BFN equilibrium LEU core was taken from this document. The MOX cores in Reference [7] provided preliminary information, which led to a second MOX core study reported in Reference [8]. The core reported in this document [8] considered one variant of the core in which the MOX fuel assembly was re-designed to utilize only four MOX fuel rod assays. It is this core from Reference [8] that has been modeled in the current study and reported in the current document.

#### **5. ASSUMPTIONS**

##### **5.1 SQN Assembly**

The conceptual fuel management design studies [5, 6] used many combinations for the number and locations of (LEU+Gd) rods and the Gd<sub>2</sub>O<sub>3</sub> concentration in these rods. A decision was taken to model the SQN MOX and LEU assemblies with 20 LEU+Gd rods.

##### **5.2 SQN Core**

For a given assembly type and a given burnup class for that assembly, nuclide concentrations were determined at a burnup value that corresponded to the average burnup of the burnup class. These nuclide concentrations were used for all burnups in the considered burnup class for that assembly.

##### **5.3 BFN Assembly**

The conceptual fuel management design studies [7, 8] provided explicit assembly loadings for the MOX and LEU assemblies in each core. The BFN MOX assemblies are well represented by a single dominant lattice zone and a single vanished lattice zone. The average enrichments in each lattice are slightly different as a result of the enrichment changes due to vanishing rods. Plenum lattices (transition zones where the short rods disappear) were not modeled. The BFN LEU assemblies had three or four lattices (in

addition to the natural uranium blankets). The major lattice in each of the dominant and vanished lattices was used to represent the entire assembly. The differences in the average enrichments were very small.

#### **5.4 BFN Core**

For a given assembly type and a given burnup class for that assembly, nuclide concentrations were determined at a burnup value that corresponded to the average burnup of the burnup class. These nuclide concentrations were used for all burnups in the considered burnup class for that assembly.

#### **5.5 Reflector Modeling**

These calculations consider infinite lattices of the specified fuel assembly. There is no reflector modeling included in the models used in this study.

#### **5.6 Various Other Assumptions**

During model development, various other assumptions were made. The assumptions are listed here with a short description/justification.

1. Lattice geometry – all data obtained from References [4 to 8] plus comments received on the Data Call.
2. All models are 2D.
3. Cross-section libraries—SCALE’s ENDF/B-VII 238-group library was the basis for all calculations. SCALE/TRITON’s “parm=weight” option was used to collapse the 238-group master library to a 49-group problem-dependent library at depletion time  $T = 0$ .
4. Resonance processing—all calculations use CENTRM/PMC [11] for resonance self-shielding. Non-gadolinium-bearing fuel rods use a lattice cell treatment in CENTRM. The gadolinium-bearing fuel rods use a multiregion treatment; the fuel region in the fuel rod is represented as five equal-area rings for the purpose of better modeling the radial depletion of gadolinium as a function of the radial distance from the rod center. Internal studies have found that five equal-area rings provide reasonable results while reducing the required computer time.
5. Dancoff factors for fuel rods were calculated for BWR assemblies, using the SCALE/MCDANCOFF module for 0, 40, and 80% void concentrations.
6. Depletion—all different fuel mixtures were depleted individually. Non-gadolinium-bearing rods were depleted by rod power, while gadolinium-bearing rods were depleted by flux. To reduce computing time, the SCALE/TRITON “assign” function was used to group rods for resonance processing purposes. Tests have shown that the use of this technique results in less than ~10 pcm of bias in the multiplication factor ( $k_{inf}$ ) compared to a “CENTRM for every rod” approach. The option addnux=4 was used in all cases. The use of this option ensures that the cross sections for those isotopes (~400) for which data exist in the 238-group ENDF/B-VII transport library are updated at each depletion step for use in the depletion solver.
7. Transport model—All fuel mixtures and structural material use  $P_1$  scattering, while all moderator/coolant mixtures use  $P_2$  scattering. Convergence criteria for the eigenvalue were set at  $10^{-5}$ . Coarse-mesh finite-difference acceleration was used on the global grid.
8. History calculations – BWR calculations were run with 25% average void in the dominant zone and 70% void in the vanished zone.
9. No control rods were defined for the PWR cases. All BWR cases were run with control blades removed.

## 6. CORE AND FUEL ASSEMBLIES DESCRIPTION

### 6.1 MOX Fuel Vectors

The MOX fuel used for both the SQN and BFN models consists of weapons-grade (WG) plutonium-amerícium mixed with depleted uranium. The Pu-Am vector is proprietary but was based on fuel used in the Catawba reactor.

### 6.2 SQN MOX Core

The SQN MOX core consists of both MOX and LEU fuel assemblies. Figure 1 presents a ¼ core map for the SQN MOX equilibrium core, showing the assembly identifiers.

	H	G	F	E	D	C	B	A	
8	03H	02B	03A	02D	03D	02F	03F	02C	
9	02B	03A	02E	03E	32D	32B	33D	02A	
10	03A	02E	03C	02E	03E	32C	33C	01G	
11	02D	03E	02E	03B	32E	03G	33B	01A	
12	03D	32D	03E	32E	32A	33E	02G		
13	02F	32B	32C	03G	33E	33A	01F		
14	03F	33D	33C	33B	02G	01C		Ass'ly ID	MOX High BU
									MOX Low BU
									LEU
15	02C	02A	01G	01A					LEU
									LEU
									LEU

**Figure 1. SQN equilibrium cycle MOX core assembly identification map.**

Table 1 summarizes the categories of the assemblies by assembly type, burnup class, average burnups in each burnup class, and fraction of each assembly category in the MOX core. The color coding of the MOX assemblies represents the two burnup classes. The color coding of the LEU assemblies represents the various U-235 enrichments.

**Table 1. Loading of the SQN equilibrium MOX core**

Assembly type	Burnup class	Number of assemblies in core	Fraction in core
MOX	1	36	0.1865
	2	36	0.1865
LEU	1	1	0.0052
	1	12	0.0622
	2	16	0.0829
	3	12	0.0622
	1	16	0.0829
	2	0	
	3	16	0.0829
	1	12	0.0622
	2	8	0.0415
	3	28	0.1451
Total		193	1.0000

### 6.3 SQN LEU Core

Figure 2 presents a  $\frac{1}{4}$  core map of the SQN equilibrium LEU core showing assembly identification numbers. Assemblies have been highlighted in different colors to clarify their location and distribution in the core.

The assemblies with higher U-235 enrichments are each categorized in three burnup classes.



Figure 2. SQN equilibrium LEU core assembly identification map.

Table 2 summarizes the categories of assemblies by enrichment (color coded), burnup class, average burnup in burnup class, and fraction of each assembly category in the LEU core.

Table 2. Loading of the SQN equilibrium LEU core

Assembly type	Burnup class	Number of assemblies in core	Fraction in core
LEU	1	1	0.0052
	1	52	0.2694
	2	28	0.1451
	3	40	0.2073
	1	28	0.1451
	2	16	0.0829
	3	28	0.1451
Total		193	1.0000

#### 6.4 BFN MOX Core

The BFN MOX core consists of both MOX and LEU fuel assemblies. Figure 3 presents a ¼ core map of the BFN equilibrium MOX core showing the assembly identifiers. Assembly identifiers contain either an “A,” “B,” “C,” or “D.” They have been highlighted in different colors to clarify their location and distribution in the core. Assemblies with “A” or “B” in their identifiers are LEU assemblies. Assemblies with “C” and “D” in their identifiers are MOX assemblies.

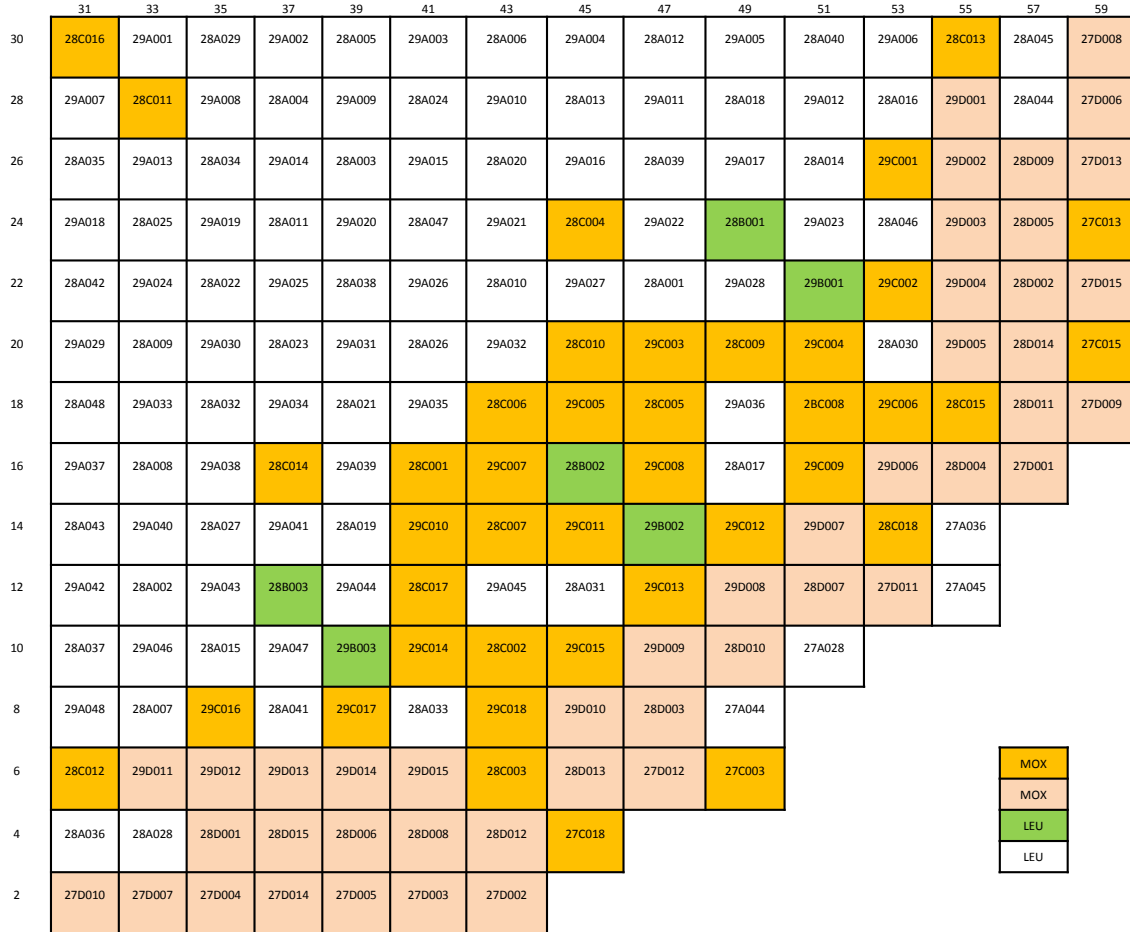


Figure 3. BFN MOX equilibrium core assay/enrichment and EOC burnup distribution.

Table 3 summarizes the categories of assemblies by assembly type, burnup class, average burnup in burnup class, and fraction of each assembly category in the MOX core. The color coding represents the assembly-average (Pu-Am) assay or the U-235 enrichment as applicable.

Table 3. Loading of the BFN MOX core

Assembly type	Burnup class	Number of assemblies in core	Fraction in core
MOX	1	72	0.0942
	2	40	0.0524
	3	48	0.0628
	1	60	0.0785
	2	52	0.0681
	3	68	0.089
LEU	1	12	0.0157
	2	0	0
	3	12	0.0157
	1	192	0.2513
	2	16	0.0209
	3	192	0.2513
Total		764	1.0000



## 6.5 BFN MOX and LEU Fuel Assemblies

The MOX and LEU assemblies in the BFN MOX core have a lower lattice (dominant) and an upper lattice (vanished). The MOX fuel assemblies have four MOX fuel rod assays. (LEU+Gd) rods have a variety of LEU enrichments, Gd<sub>2</sub>O<sub>3</sub> concentrations, number and placements of Gd fuel rods. These variations were all modeled explicitly. The locations of the part-length rods are the same for all assemblies (MOX and LEU) in the BFN MOX core. Fuel assemblies typically have natural uranium fuel blankets at the top and bottom, as well as a plenum region where short rods are transitioning out. These regions are not modeled in this study.

## 6.6 BFN LEU Core

Figure 4 presents a 1/4 core map of the BFN LEU equilibrium core showing the assembly identifiers which contain either an “A,” “B,” or “C”. They have been highlighted in different colors to clarify their location and distribution in the core.

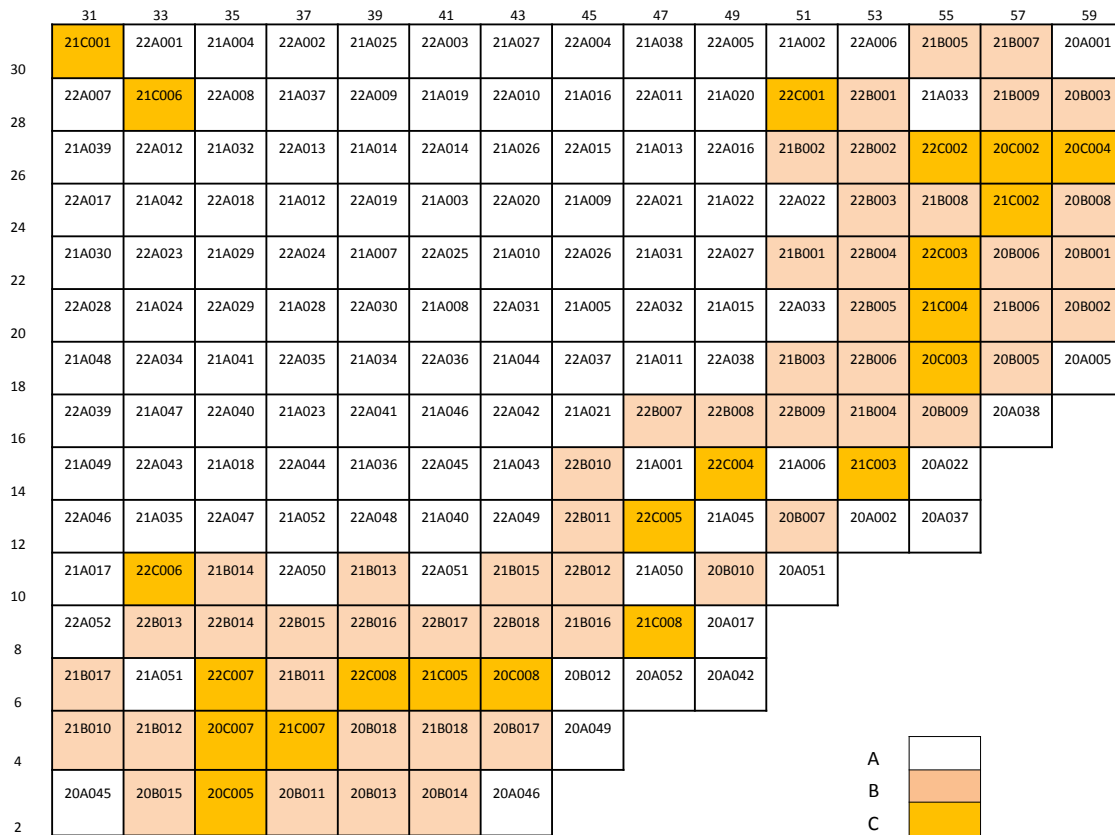


Figure 4. BFN equilibrium EOC burnup distribution in LEU core (GWd/MTHM).

Table 4 summarizes the categories of LEU assemblies by enrichments, average burnups in each burnup class, and the fraction in the core of each LEU assembly category.

**Table 4. Loading of BFN equilibrium LEU core**

Assembly type	Burnup classes	Number of assemblies in core	Fraction in core
LEU	1	208	0.2723
	2	20	0.0262
	3	240	0.3141
	1	72	0.0942
	2	56	0.0733
	3	80	0.1047
	1	36	0.0471
	2	24	0.0314
	3	28	0.0367
Total		764	1.0000

## 7. BURNUP STEPS

As previously mentioned, a maximum exposure of approximately 60 GWd/MTHM was chosen. The SQN fuel was depleted for 1700 days to reach a total burnup of 66.300 GWd/MTHM. The BFN fuel was depleted for 2102 days to reach a total burnup of 60.958 GWd/MTHM. The burnup steps used can be found in Table 5.

**Table 5. Burnup steps**

SQN				BFN			
Time (days)	Burnup (GWd/MTHM)	Time (days)	Burnup (GWd/MTHM)	Time (days)	Burnup (GWd/MTHM)	Time (days)	Burnup (GWd/MTHM)
0	0.000	875	34.125	0	0.00	1052	30.508
50	1.950	950	37.050	2	0.058	1202	34.858
125	4.875	1025	39.975	92	2.668	1352	39.208
200	7.800	1100	42.900	182	5.278	1502	43.558
275	10.725	1175	45.825	272	7.888	1652	47.908
350	13.650	1250	48.750	362	10.498	1802	52.258
425	16.575	1325	51.675	452	13.108	1952	56.608
500	19.500	1400	54.600	542	15.718	2102	60.958
575	22.425	1475	57.525	632	18.328		
650	25.350	1550	60.450	722	20.938		
725	28.275	1625	63.375	812	23.548		
800	31.200	1700	66.300	902	26.158		

## 8. CALCULATION PROCEDURE

The computational analysis of the measurements was carried out using the two-dimensional (2D) depletion sequence T-DEPL of the TRITON module in the SCALE computer code system (version 6.1). The T-DEPL sequence in TRITON couples the 2D arbitrary polygonal mesh, discrete-ordinates transport code NEWT with the depletion and decay code ORIGEN-S in order to perform the burnup simulation. At each depletion step, the transport flux solution from NEWT is used to generate cross sections and assembly power distributions for the ORIGEN-S calculations; the isotopic composition data resulting

from ORIGEN-S is employed in the subsequent transport calculation to obtain cross sections and power distributions for the next depletion step in an iterative manner throughout the irradiation history.

TRITON has the capability of simulating the depletion of multiple mixtures in a fuel assembly model. This is a very useful and powerful feature in a nuclide inventory analysis, as it allows a more appropriate representation of the local flux distribution and neutronic environment for a specific measured fuel rod in the assembly. The flux normalization in a TRITON calculation can be performed using, as a basis, the power in a specified mixture, the total power corresponding to multiple mixtures, or the assembly power. An average assembly power was used for these calculations.

Previous work [9, 10] has demonstrated the necessity to calculate individual fuel rod Dancoff factors when simulating highly heterogeneous BWR fuel assemblies. The built-in Dancoff factor routine in MIPLIB assumes a uniform distribution of uniform fuel rods, an assumption that is not appropriate in these cases. It was found that, for the ATRIUM 10 fuel assembly, the MCDANCOFF-calculated Dancoff factors were 25–30% lower than SCALE/MIPLIB-calculated infinite lattice Dancoff factors for edge fuel rods and were up to 20% lower for some interior rods.

ORNL used the SCALE module MCDANCOFF and KENO-VI input files for the ATRIUM 10 assembly to calculate Dancoff factors for each pin in the dominant and vanished lattices for 0 wt%, 40 wt%, and 80 wt% void concentrations.

The MCDANCOFF Dancoff factors are applied in the TRITON model during the CENTRM cross section processing step by using the dan2pitch(N) fuel option, where N is the fuel material identifier number whose Dancoff factor is being modified.

The W17 HTP assembly lattice is sufficiently homogeneous in a geometrical sense, and the Dancoff factors calculated by MIPLIB are acceptable.

All TRITON calculations employed the SCALE 238-group cross-section library based on ENDF/B-VII data collapsed to 49 groups (“weight” option in TRITON parm statement), for the pin-cell cross section treatment. Default values were used for the convergence parameters in the NEWT transport calculation.

## 9. RESULTS

### 9.1 Dancoff Factors

Dancoff factors were determined based on data obtained from SCALE/MCDANCOFF simulations as previously discussed in this report. The Dancoff factors corresponding to 25 vol % and 70 vol % void values were calculated from the fitting of the data calculated with MCDANCOFF for three pre-calculated sets at void values (0, 40, and 80 vol %). To reduce the number of self-shielding calculations in the depletion simulation, a single value was used for the Dancoff factors for edge rods (the corner rods excluded), which was calculated as an average over the values for all edge rods; this assumption was used since there is little difference among the individual calculated Dancoff factors for the edge rods. The use of an average Dancoff factor for the corner rods was determined to be inappropriate, as the Dancoff factors for the corner rods are significantly different one from the other; therefore, individual Dancoff factors were used for the corner rods.

## 9.2 Multiplication Factor ( $k$ .)

### SQN

Figure 5 presents the infinite medium multiplication factor ( $k_{\infty}$ ) trajectories for LEU fuel assemblies with different enrichments that are present in the SQN LEU equilibrium core. Figure 6 presents the  $k_{\infty}$  trajectories for the LEU and MOX fuel assemblies present in the SQN equilibrium MOX core. The rise in reactivity (up to approximately 8 GWd/MTHM in this case) is characteristic of assemblies that include gadolinia rods, resulting from the burnout of the gadolinium. Once the gadolinium has been consumed, the reactivity decreases steadily as fissile material is consumed at a rate greater than it is created.

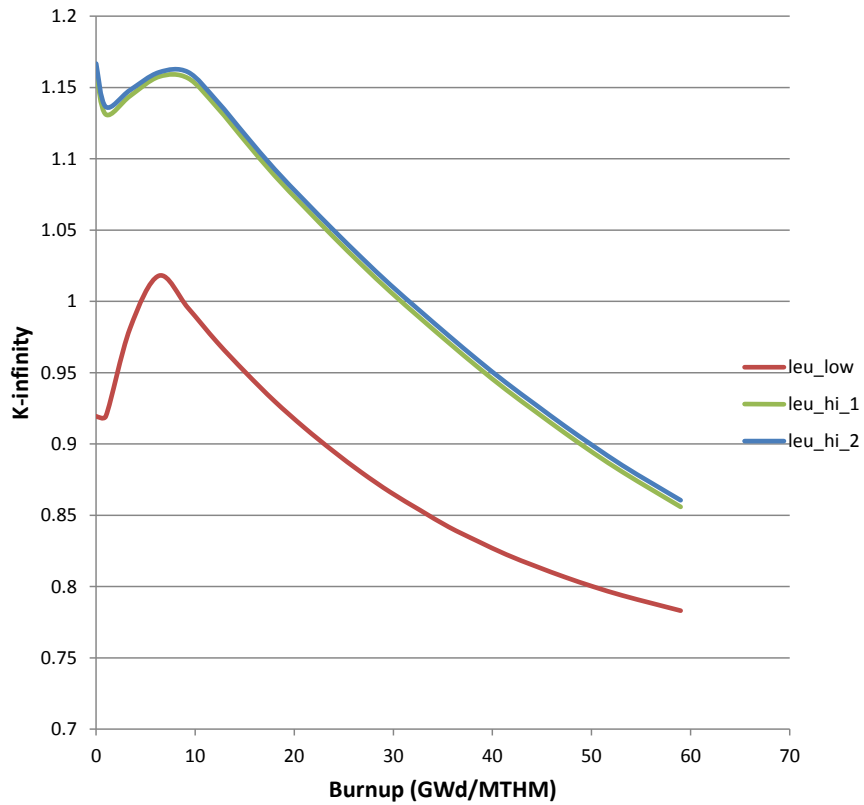


Figure 5.  $k_{\infty}$  trajectories for SQN LEU core assemblies.

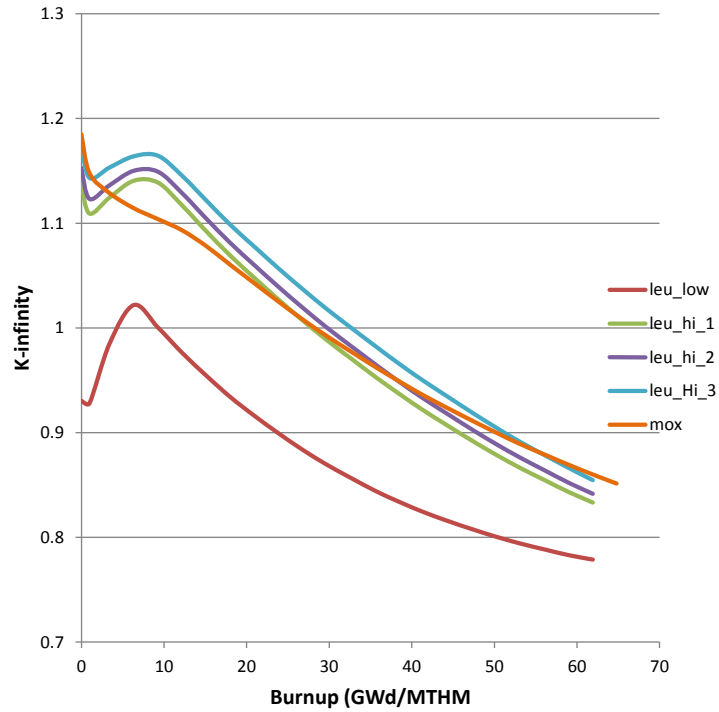


Figure 6.  $k_{\infty}$  trajectories for SQN MOX core assemblies.

**BFN**

Figure 7 and Figure 8 present the  $k_{\infty}$  trajectories of the dominant and vanished zones of the three LEU assemblies in the BFN equilibrium LEU core. The reactivity peaks occur at higher burnup than for the SQN assemblies due to higher loading of gadolinia.

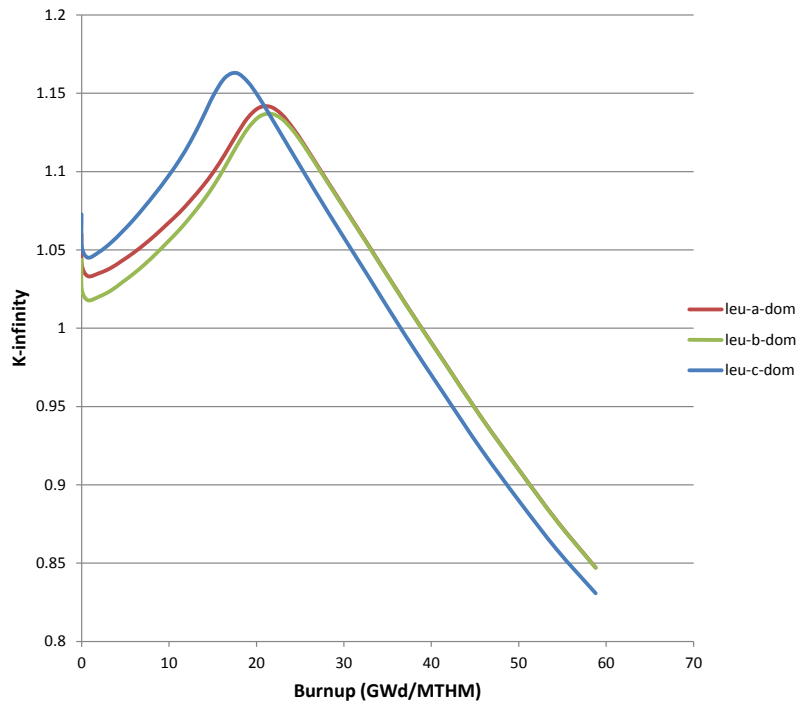


Figure 7.  $k_{\infty}$  trajectories for BFN LEU core assemblies – dominant zone.

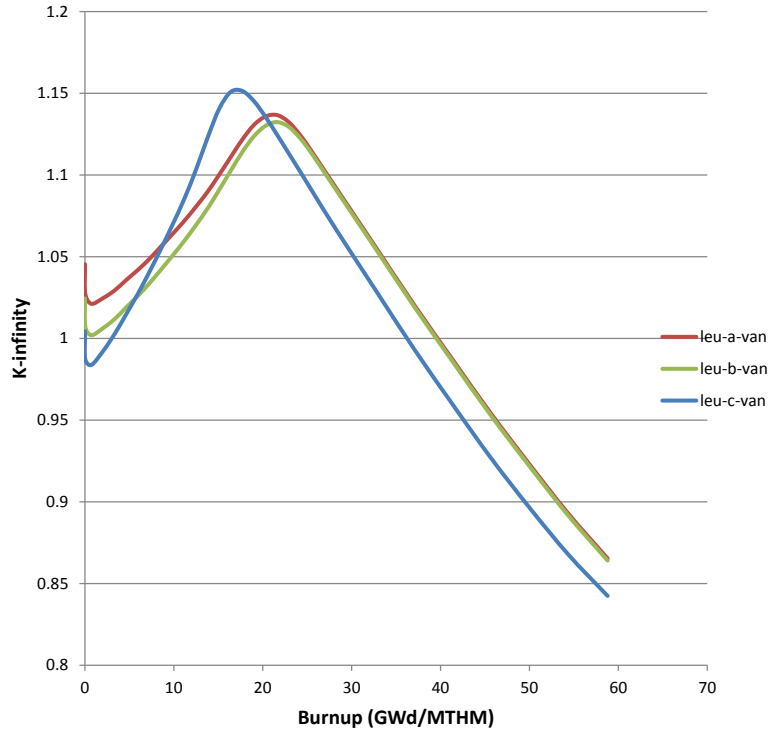


Figure 8.  $k_{\infty}$  trajectories for BFN LEU core assemblies – vanished zone.

Figure 9 and Figure 10 present the  $k_{\infty}$  trajectories of the dominant and vanished zones of the two LEU assemblies and the two MOX assemblies in the BFN equilibrium MOX core.

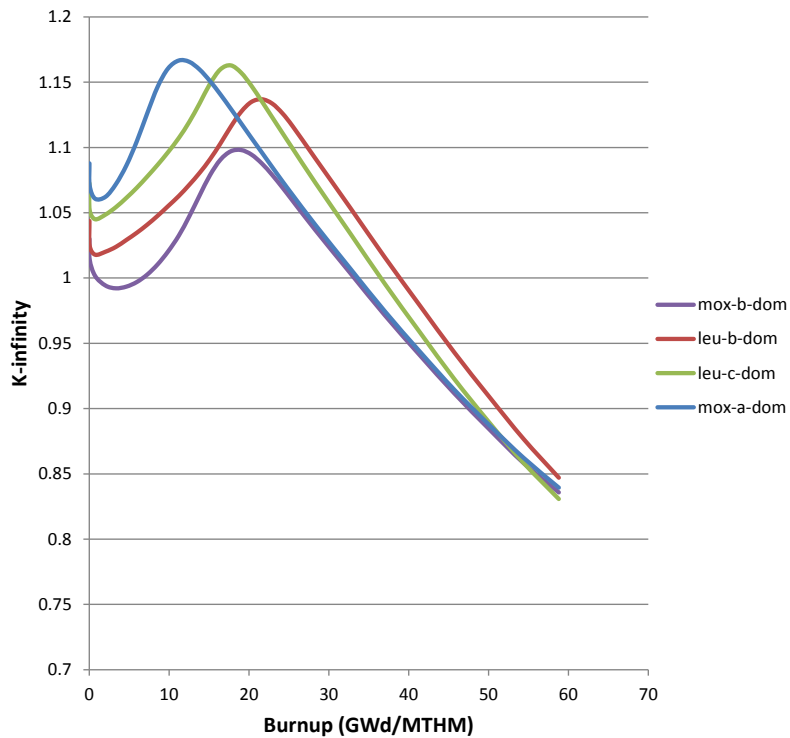


Figure 9.  $k_{\infty}$  trajectories for BFN MOX core assemblies – dominant zone.

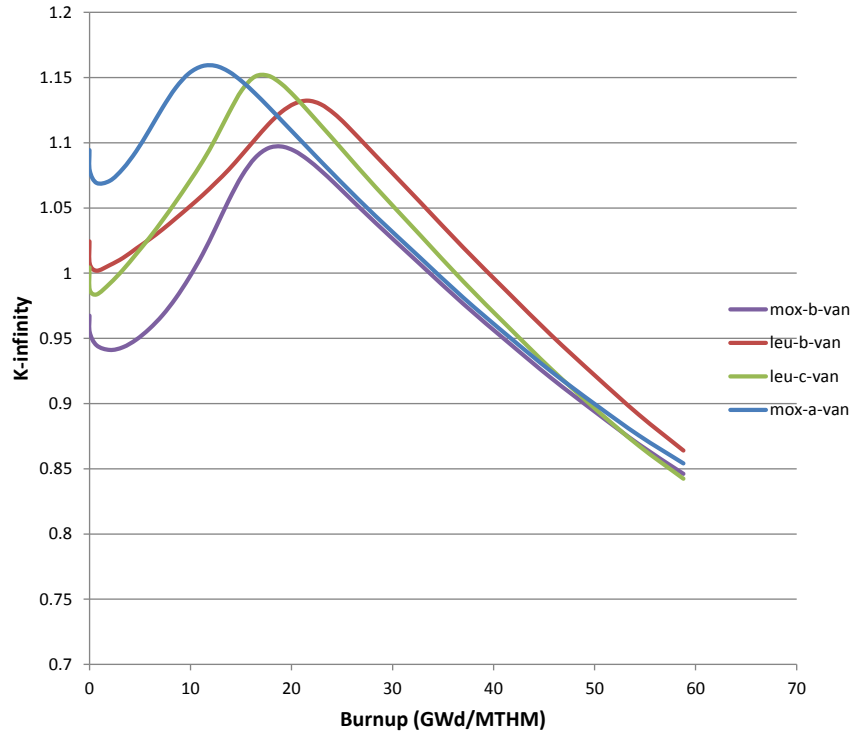


Figure 10. k. trajectories for BFN MOX core assemblies – vanished zone.

### 9.3 Isotopic Masses and Isotopic Ratios

#### SQN

Table 6 presents a summary of the total mass of each of the considered nuclides in the SQN MOX and LEU cores as well as the ratios of the nuclide contents in the MOX core and the LEU core. The total isotopic masses in a core were determined by multiplying the core-weighted-average isotopic mass per MTHM by the number of tonnes in each core (87.302 MTHM in the MOX core, 87.921 MTHM in the LEU core).

Table 6. Summary of nuclide concentrations for the SQN equilibrium MOX and LEU cores

Isotope	MOX core			Total mass <sup>a</sup> (g)	LEU core		Ratios
	Core-weighted-average components				Core-weighted average of LEU assemblies	Total mass <sup>a</sup> (g)	
	MOX assemblies	LEU assemblies	(MOX+LEU) assemblies				
	(g/MTHM)	(g/MTHM)	(g/MTHM)		(g/MTHM)	(g)	
am241	6.1300E+01	2.8995E+01	9.0294E+01	7.8828E+03	4.4701E+01	3.9302E+03	2.0199
am242	1.1532E-01	7.5635E-02	1.9096E-01	1.6671E+01	1.1608E-01	1.0206E+01	1.6450
am242m	1.0313E+00	4.3809E-01	1.4694E+00	1.2828E+02	6.7310E-01	5.9180E+01	2.1830
am243	9.3154E+01	8.3508E+01	1.7666E+02	1.5423E+04	1.2962E+02	1.1396E+04	1.3630
am244	5.2949E-03	5.3120E-03	1.0607E-02	9.2600E-01	8.2218E-03	7.2287E-01	1.2901
am245	1.3786E-06	1.8349E-06	3.2135E-06	2.8054E-04	2.8557E-06	2.5108E-04	1.1253
ba137m	7.6202E-05	1.4121E-04	2.1741E-04	1.8980E-02	2.2221E-04	1.9537E-02	0.9784
ba139	4.1130E-02	7.2602E-02	1.1373E-01	9.9289E+00	1.1616E-01	1.0213E+01	0.9791
ba140	8.7379E+00	1.5551E+01	2.4289E+01	2.1205E+03	2.4893E+01	2.1886E+03	0.9757

Table 6 (continued)

Isotope	MOX core				LEU core		Ratios
	Core-weighted-average components			Total mass <sup>a</sup>	Core-weighted average of LEU assemblies	Total mass <sup>a</sup>	
	MOX assemblies	LEU assemblies	(MOX+LEU) assemblies				
	(g/MTHM)	(g/MTHM)	(g/MTHM)	(g)	(g/MTHM)	(g)	
ba141	8.1720E-03	1.4570E-02	2.2742E-02	1.9854E+00	2.3322E-02	2.0505E+00	0.9751
ba142	4.3390E-03	7.9639E-03	1.2303E-02	1.0741E+00	1.2766E-02	1.1224E+00	0.9637
br83	2.2175E-03	4.8823E-03	7.0998E-03	6.1983E-01	7.8858E-03	6.9332E-01	0.9003
br84	7.8295E-04	1.8328E-03	2.6157E-03	2.2836E-01	2.9658E-03	2.6075E-01	0.8820
br84m	1.1391E-05	1.5663E-05	2.7054E-05	2.3619E-03	2.4757E-05	2.1767E-03	1.0928
br85	8.9117E-05	2.2458E-04	3.1370E-04	2.7387E-02	3.6453E-04	3.2050E-02	0.8606
br87	4.0578E-05	1.0709E-04	1.4767E-04	1.2892E-02	1.7402E-04	1.5300E-02	0.8486
cd112	9.2797E+00	8.7546E+00	1.8034E+01	1.5744E+03	1.3619E+01	1.1974E+03	1.3242
cd114	1.0056E+01	1.0745E+01	2.0802E+01	1.8160E+03	1.6747E+01	1.4724E+03	1.2421
ce140	4.5240E+02	9.4872E+02	1.4011E+03	1.2232E+05	1.4939E+03	1.3135E+05	0.9379
ce141	2.1070E+01	3.7491E+01	5.8561E+01	5.1125E+03	6.0022E+01	5.2772E+03	0.9757
ce142	4.0173E+02	8.5565E+02	1.2574E+03	1.0977E+05	1.3485E+03	1.1856E+05	0.9325
ce143	7.9138E-01	1.4946E+00	2.2859E+00	1.9957E+02	2.3990E+00	2.1093E+02	0.9529
ce144	1.2551E+02	2.6729E+02	3.9280E+02	3.4292E+04	4.2486E+02	3.7354E+04	0.9245
ce145	8.4621E-04	1.5723E-03	2.4185E-03	2.1114E-01	2.5212E-03	2.2166E-01	0.9593
cm242	1.6054E+01	1.0081E+01	2.6134E+01	2.2816E+03	1.5537E+01	1.3660E+03	1.6821
cm243	4.4408E-01	2.5586E-01	6.9994E-01	6.1106E+01	3.9923E-01	3.5101E+01	1.7532
cm244	3.6855E+01	3.2888E+01	6.9744E+01	6.0887E+03	5.2244E+01	4.5934E+03	1.3350
cm245	3.2661E+00	2.2720E+00	5.5382E+00	4.8349E+02	3.6108E+00	3.1747E+02	1.5338
cm246	2.4401E-01	2.3291E-01	4.7692E-01	4.1636E+01	3.8626E-01	3.3961E+01	1.2347
co58	5.0661E-18	4.1519E-18	9.2180E-18	8.0475E-16	6.9667E-18	6.1252E-16	1.3232
co60	3.3644E-15	7.7392E-15	1.1104E-14	9.6936E-13	1.2012E-14	1.0561E-12	0.9244
cs133	4.4752E+02	8.3136E+02	1.2789E+03	1.1165E+05	1.3097E+03	1.1515E+05	0.9765
cs134	5.2559E+01	1.0487E+02	1.5743E+02	1.3744E+04	1.6345E+02	1.4370E+04	0.9632
cs135	2.5267E+02	3.1145E+02	5.6411E+02	4.9248E+04	4.9445E+02	4.3473E+04	1.1409
cs136	3.4491E-01	4.5566E-01	8.0056E-01	6.9890E+01	7.1609E-01	6.2959E+01	1.1180
cs137	4.9515E+02	9.2049E+02	1.4156E+03	1.2359E+05	1.4485E+03	1.2735E+05	0.9773
cs138	1.7280E-02	3.0422E-02	4.7702E-02	4.1645E+00	4.8672E-02	4.2793E+00	0.9801
cs139	4.4586E-03	7.9176E-03	1.2376E-02	1.0805E+00	1.2673E-02	1.1142E+00	0.9766
cs140	4.0784E-04	7.6409E-04	1.1719E-03	1.0231E-01	1.2253E-03	1.0773E-01	0.9565
cs141	1.3302E-04	2.3821E-04	3.7123E-04	3.2409E-02	3.8123E-04	3.3518E-02	0.9738
eul51	1.5693E-02	1.0511E-02	2.6203E-02	2.2876E+00	1.7013E-02	1.4958E+00	1.5402
eul53	5.8823E+01	8.5814E+01	1.4464E+02	1.2627E+04	1.3371E+02	1.1756E+04	1.0817
eul54	1.6079E+01	2.0091E+01	3.6170E+01	3.1577E+03	3.1108E+01	2.7350E+03	1.1627
eul55	4.4360E+00	6.1459E+00	1.0582E+01	9.2382E+02	9.5470E+00	8.3938E+02	1.1084
gd154	1.1160E+01	1.7226E+01	2.8386E+01	2.4782E+03	2.7593E+01	2.4260E+03	1.0288
gd155	1.3644E-01	1.4415E-01	2.8058E-01	2.4495E+01	2.2953E-01	2.0180E+01	1.2224
gd156	2.2693E+02	3.6597E+02	5.9290E+02	5.1761E+04	5.8283E+02	5.1243E+04	1.0173
gd157	2.0291E-01	1.8963E-01	3.9253E-01	3.4269E+01	3.0097E-01	2.6461E+01	1.3042
gd158	2.4452E+02	3.9724E+02	6.4176E+02	5.6027E+04	6.3295E+02	5.5650E+04	1.0139
il29	8.9571E+01	1.1527E+02	2.0484E+02	1.7883E+04	1.8048E+02	1.5868E+04	1.1350
il30	3.9876E-03	6.7118E-03	1.0699E-02	9.3407E-01	1.0430E-02	9.1705E-01	1.0258
il31	3.3507E+00	5.3226E+00	8.6733E+00	7.5720E+02	8.4738E+00	7.4502E+02	1.0236
il32	5.7863E-02	9.3432E-02	1.5130E-01	1.3208E+01	1.4885E-01	1.3087E+01	1.0164
il33	7.0339E-01	1.1884E+00	1.8918E+00	1.6515E+02	1.8975E+00	1.6683E+02	0.9970



Table 6 (continued)

Isotope	MOX core				LEU core		Ratios
	Core-weighted-average components			Total mass <sup>a</sup>	Core-weighted average of LEU assemblies	Total mass <sup>a</sup>	
	MOX assemblies	LEU assemblies	(MOX+LEU) assemblies				
	(g/MTHM)	(g/MTHM)	(g/MTHM)	(g)	(g/MTHM)	(g)	
i134	3.2549E-02	5.6243E-02	8.8791E-02	7.7516E+00	8.9898E-02	7.9039E+00	0.9877
i135	2.1708E-01	3.6386E-01	5.8095E-01	5.0718E+01	5.8065E-01	5.1051E+01	1.0005
i136m	1.0945E-04	1.6951E-04	2.7896E-04	2.4354E-02	2.6944E-04	2.3690E-02	1.0353
kr83m	1.6814E-03	3.7055E-03	5.3870E-03	4.7029E-01	5.9831E-03	5.2604E-01	0.9004
kr85	7.0252E+00	2.3627E+01	3.0652E+01	2.6760E+03	3.7385E+01	3.2869E+03	0.8199
kr85m	7.9709E-03	2.0104E-02	2.8075E-02	2.4510E+00	3.2631E-02	2.8689E+00	0.8604
kr87	4.3830E-03	1.1203E-02	1.5586E-02	1.3607E+00	1.8190E-02	1.5992E+00	0.8569
kr88	1.2716E-02	3.3429E-02	4.6145E-02	4.0285E+00	5.4331E-02	4.7768E+00	0.8493
kr89	2.8267E-04	7.7710E-04	1.0598E-03	9.2520E-02	1.2647E-03	1.1119E-01	0.8380
kr90	4.6862E-05	1.3857E-04	1.8543E-04	1.6189E-02	2.2595E-04	1.9865E-02	0.8207
la139	4.5092E+02	9.2486E+02	1.3758E+03	1.2011E+05	1.4568E+03	1.2809E+05	0.9444
la140	1.2379E+00	2.2228E+00	3.4607E+00	3.0212E+02	3.5558E+00	3.1263E+02	0.9733
la141	1.0624E-01	1.8893E-01	2.9517E-01	2.5769E+01	3.0245E-01	2.6591E+01	0.9759
la142	3.8986E-02	7.0749E-02	1.0973E-01	9.5800E+00	1.1336E-01	9.9670E+00	0.9680
la143	5.6328E-03	1.0624E-02	1.6257E-02	1.4193E+00	1.7054E-02	1.4994E+00	0.9533
mo100	3.7971E+02	7.0809E+02	1.0878E+03	9.4966E+04	1.1144E+03	9.7977E+04	0.9761
mo92	2.9870E-12	5.4952E-13	3.5365E-12	3.0875E-10	8.9063E-13	7.8305E-11	3.9709
mo94	3.8048E-03	8.0906E-03	1.1895E-02	1.0385E+00	1.2711E-02	1.1175E+00	0.9359
mo95	2.0992E+02	4.9562E+02	7.0553E+02	6.1594E+04	7.7983E+02	6.8564E+04	0.9047
mo96	1.2057E+01	3.2275E+01	4.4331E+01	3.8702E+03	5.0617E+01	4.4503E+03	0.8758
mo97	2.9777E+02	6.1137E+02	9.0914E+02	7.9369E+04	9.6321E+02	8.4686E+04	0.9439
mo98	3.2142E+02	6.2488E+02	9.4630E+02	8.2614E+04	9.8383E+02	8.6499E+04	0.9619
mo99	1.4926E+00	2.5429E+00	4.0355E+00	3.5231E+02	4.0613E+00	3.5707E+02	0.9936
nb95	1.4096E+01	2.7616E+01	4.1712E+01	3.6415E+03	4.4357E+01	3.8999E+03	0.9404
nb97	2.3250E-02	4.1477E-02	6.4727E-02	5.6508E+00	6.6389E-02	5.8370E+00	0.9750
nb97m	5.4287E-07	6.7889E-07	1.2218E-06	1.0666E-04	1.0672E-06	9.3826E-05	1.1449
nd142	5.5673E+00	1.5360E+01	2.0927E+01	1.8270E+03	2.4106E+01	2.1195E+03	0.8681
nd143	2.9757E+02	6.0438E+02	9.0195E+02	7.8742E+04	9.5495E+02	8.3960E+04	0.9445
nd144	2.6138E+02	7.2314E+02	9.8452E+02	8.5950E+04	1.1349E+03	9.9779E+04	0.8675
nd145	2.3041E+02	5.0147E+02	7.3188E+02	6.3895E+04	7.9128E+02	6.9570E+04	0.9249
nd146	2.4484E+02	5.3753E+02	7.8237E+02	6.8302E+04	8.4610E+02	7.4390E+04	0.9247
nd147	3.0168E+00	5.2619E+00	8.2787E+00	7.2275E+02	8.4139E+00	7.3976E+02	0.9839
nd148	1.4519E+02	2.8070E+02	4.2589E+02	3.7181E+04	4.4171E+02	3.8836E+04	0.9642
nd150	8.4858E+01	1.3318E+02	2.1804E+02	1.9035E+04	2.0906E+02	1.8381E+04	1.0429
np237	6.1198E+01	3.5749E+02	4.1869E+02	3.6553E+04	5.5991E+02	4.9228E+04	0.7478
np238	1.6589E-01	1.1191E+00	1.2850E+00	1.1218E+02	1.7405E+00	1.5303E+02	0.7383
np239	3.3225E+01	5.9415E+01	9.2641E+01	8.0877E+03	9.3324E+01	8.2051E+03	0.9927
np240	3.8372E-04	7.6144E-04	1.1452E-03	9.9975E-02	1.1795E-03	1.0370E-01	0.9709
pd105	2.8790E+02	2.8550E+02	5.7340E+02	5.0059E+04	4.4519E+02	3.9142E+04	1.2880
pd107	1.8436E+02	1.5620E+02	3.4056E+02	2.9732E+04	2.4237E+02	2.1309E+04	1.4052
pd108	1.2663E+02	1.0316E+02	2.2979E+02	2.0061E+04	1.5999E+02	1.4066E+04	1.4363
pd110	4.1244E+01	3.4322E+01	7.5566E+01	6.5970E+03	5.3252E+01	4.6820E+03	1.4190
pm147	7.0233E+01	1.3400E+02	2.0423E+02	1.7830E+04	2.1317E+02	1.8742E+04	0.9580
pr143	7.5340E+00	1.4231E+01	2.1765E+01	1.9001E+03	2.2842E+01	2.0083E+03	0.9528
pr144	5.3155E-03	1.1329E-02	1.6645E-02	1.4531E+00	1.8007E-02	1.5832E+00	0.9243

Table 6 (continued)

Isotope	MOX core			LEU core			Ratios
	Core-weighted-average components			Total mass <sup>a</sup>	Core-weighted average of LEU assemblies	Total mass <sup>a</sup>	
	MOX assemblies	LEU assemblies	(MOX+LEU) assemblies				
	(g/MTHM)	(g/MTHM)	(g/MTHM)	(g)	(g/MTHM)	(g)	
pr145	1.0107E-01	1.8769E-01	2.8876E-01	2.5209E+01	3.0097E-01	2.6461E+01	0.9594
pu237	1.8641E-04	3.8721E-04	5.7362E-04	5.0078E-02	6.0514E-04	5.3204E-02	0.9479
pu238	4.2724E+01	1.3320E+02	1.7592E+02	1.5358E+04	2.0818E+02	1.8303E+04	0.8450
pu239	6.1164E+03	4.0054E+03	1.0122E+04	8.8365E+05	6.3346E+03	5.5695E+05	1.5979
pu240	2.9011E+03	1.5056E+03	4.4067E+03	3.8471E+05	2.3371E+03	2.0548E+05	1.8855
pu241	1.5081E+03	9.5997E+02	2.4680E+03	2.1546E+05	1.4833E+03	1.3041E+05	1.6639
pu242	3.9209E+02	3.4373E+02	7.3582E+02	6.4239E+04	5.3001E+02	4.6599E+04	1.3883
pu243	1.0501E-01	1.0713E-01	2.1214E-01	1.8520E+01	1.6384E-01	1.4405E+01	1.2947
pu244	2.6306E-02	2.5456E-02	5.1762E-02	4.5189E+00	4.0133E-02	3.5285E+00	1.2898
pu245	1.8115E-06	1.7834E-06	3.5949E-06	3.1384E-04	2.8135E-06	2.4736E-04	1.2778
rb86	6.8339E-03	2.3476E-02	3.0310E-02	2.6461E+00	3.6784E-02	3.2341E+00	0.8240
rb88	1.3703E-03	3.5560E-03	4.9263E-03	4.3007E-01	5.7762E-03	5.0785E-01	0.8529
rb89	1.5291E-03	4.0266E-03	5.5556E-03	4.8502E-01	6.5446E-03	5.7540E-01	0.8489
rb90	2.4768E-04	7.1139E-04	9.5908E-04	8.3729E-02	1.1589E-03	1.0189E-01	0.8276
rb90m	1.1878E-04	2.2910E-04	3.4788E-04	3.0370E-02	3.6845E-04	3.2395E-02	0.9442
rb91	1.2848E-04	3.2434E-04	4.5282E-04	3.9532E-02	5.2635E-04	4.6277E-02	0.8603
rh103m	2.3771E-02	3.2519E-02	5.6290E-02	4.9142E+00	5.1333E-02	4.5132E+00	1.0966
rh105	7.0353E-01	8.3950E-01	1.5430E+00	1.3471E+02	1.3159E+00	1.1569E+02	1.1726
rh105m	7.0777E-05	8.6320E-05	1.5710E-04	1.3715E-02	1.3512E-04	1.1880E-02	1.1626
rh106	1.1839E-04	1.2207E-04	2.4047E-04	2.0993E-02	1.8916E-04	1.6631E-02	1.2713
rh107	4.7737E-03	5.3367E-03	1.0110E-02	8.8266E-01	8.3041E-03	7.3011E-01	1.2175
ru101	3.2958E+02	5.8372E+02	9.1330E+02	7.9732E+04	9.1828E+02	8.0736E+04	0.9946
ru102	3.7285E+02	6.0332E+02	9.7618E+02	8.5222E+04	9.4740E+02	8.3296E+04	1.0304
ru103	2.4218E+01	3.3134E+01	5.7352E+01	5.0069E+03	5.2303E+01	4.5986E+03	1.0965
ru104	3.5050E+02	4.0936E+02	7.5986E+02	6.6337E+04	6.3972E+02	5.6245E+04	1.1878
ru105	9.3554E-02	1.1411E-01	2.0766E-01	1.8129E+01	1.7865E-01	1.5707E+01	1.1624
ru106	1.1860E+02	1.1620E+02	2.3480E+02	2.0498E+04	1.8011E+02	1.5836E+04	1.3036
ru107	8.1240E-04	9.0834E-04	1.7207E-03	1.5022E-01	1.4131E-03	1.2424E-01	1.2177
sb125	5.3550E+00	6.5719E+00	1.1927E+01	1.0412E+03	1.0265E+01	9.0250E+02	1.1619
sb127	1.8097E-01	2.4298E-01	4.2396E-01	3.7012E+01	3.8330E-01	3.3700E+01	1.1061
sb129	2.5298E-02	3.5388E-02	6.0685E-02	5.2979E+00	5.5926E-02	4.9171E+00	1.0851
sb130	3.2638E-03	5.1681E-03	8.4318E-03	7.3611E-01	8.2242E-03	7.2308E-01	1.0252
sb130m	5.1785E-04	8.5985E-04	1.3777E-03	1.2027E-01	1.3717E-03	1.2060E-01	1.0043
sb133	3.6134E-04	7.0257E-04	1.0639E-03	9.2881E-02	1.1273E-03	9.9112E-02	0.9438
se84	7.2983E-05	1.7326E-04	2.4624E-04	2.1497E-02	2.8050E-04	2.4662E-02	0.8779
sm147	2.4059E+01	5.1192E+01	7.5251E+01	6.5695E+03	8.0481E+01	7.0759E+03	0.9350
sm148	5.2764E+01	1.0460E+02	1.5736E+02	1.3738E+04	1.6368E+02	1.4391E+04	0.9614
sm149	1.7680E+00	1.8402E+00	3.6083E+00	3.1501E+02	2.9663E+00	2.6080E+02	1.2164
sm150	1.1851E+02	2.1971E+02	3.3823E+02	2.9528E+04	3.4468E+02	3.0305E+04	0.9813
sm151	8.7913E+00	9.1911E+00	1.7982E+01	1.5699E+03	1.4658E+01	1.2888E+03	1.2268
sm152	5.0743E+01	7.2776E+01	1.2352E+02	1.0783E+04	1.1474E+02	1.0088E+04	1.0765
sm154	2.3564E+01	2.7304E+01	5.0867E+01	4.4408E+03	4.2618E+01	3.7470E+03	1.1936
sn130	1.7041E-04	3.0586E-04	4.7627E-04	4.1579E-02	4.8943E-04	4.3031E-02	0.9731
sr89	7.4778E+00	2.0159E+01	2.7637E+01	2.4127E+03	3.2752E+01	2.8796E+03	0.8438
sr90	1.1536E+02	4.1317E+02	5.2853E+02	4.6142E+04	6.5380E+02	5.7483E+04	0.8084

Table 6 (continued)

Isotope	MOX core			LEU core			Ratios
	Core-weighted-average components			Total mass <sup>a</sup>	Core-weighted average of LEU assemblies	Total mass <sup>a</sup>	
	MOX assemblies	LEU assemblies	(MOX+LEU) assemblies				
	(g/MTHM)	(g/MTHM)	(g/MTHM)	(g)	(g/MTHM)	(g)	
sr91	8.5443E-02	2.0755E-01	2.9300E-01	2.5579E+01	3.3644E-01	2.9580E+01	0.8709
sr92	2.7899E-02	6.3120E-02	9.1019E-02	7.9461E+00	1.0204E-01	8.9714E+00	0.8920
sr93	1.5747E-03	3.3319E-03	4.9066E-03	4.2835E-01	5.3734E-03	4.7244E-01	0.9131
sr94	2.7056E-04	5.6272E-04	8.3327E-04	7.2746E-02	9.0692E-04	7.9737E-02	0.9188
tc101	5.3720E-03	8.7257E-03	1.4098E-02	1.2308E+00	1.3903E-02	1.2224E+00	1.0140
tc99	3.1008E+02	5.9221E+02	9.0230E+02	7.8772E+04	9.3290E+02	8.2021E+04	0.9672
tc99m	1.2038E-01	2.0518E-01	3.2556E-01	2.8422E+01	3.2766E-01	2.8808E+01	0.9936
te125m	6.6803E-02	8.1107E-02	1.4791E-01	1.2913E+01	1.2650E-01	1.1122E+01	1.1693
te127	1.7982E-02	2.3948E-02	4.1930E-02	3.6606E+00	3.7786E-02	3.3222E+00	1.1097
te127m	1.5741E-01	1.6505E-01	3.2246E-01	2.8152E+01	2.6231E-01	2.3062E+01	1.2293
te129	6.6784E-03	9.3447E-03	1.6023E-02	1.3988E+00	1.4766E-02	1.2982E+00	1.0852
te129m	2.3539E-03	2.9287E-03	5.2826E-03	4.6118E-01	4.5856E-03	4.0317E-01	1.1520
te131	6.2990E-03	1.0290E-02	1.6589E-02	1.4482E+00	1.6401E-02	1.4419E+00	1.0115
te131m	9.0758E-02	1.2060E-01	2.1136E-01	1.8452E+01	1.9027E-01	1.6729E+01	1.1108
te132	1.8682E+00	3.0448E+00	4.9130E+00	4.2892E+02	4.8530E+00	4.2668E+02	1.0124
te133	3.8733E-03	6.8119E-03	1.0685E-02	9.3284E-01	1.0895E-02	9.5793E-01	0.9807
te133m	1.2773E-02	2.2009E-02	3.4782E-02	3.0365E+00	3.5189E-02	3.0939E+00	0.9884
te134	1.8644E-02	3.4975E-02	5.3618E-02	4.6810E+00	5.6102E-02	4.9325E+00	0.9557
u234	6.9425E+00	1.3601E+02	1.4295E+02	1.2480E+04	2.2547E+02	1.9823E+04	0.6340
u235	7.8336E+02	7.9531E+03	8.7364E+03	7.6271E+05	1.3630E+04	1.1984E+06	0.6410
u236	1.8303E+02	3.4053E+03	3.5883E+03	3.1327E+05	5.4343E+03	4.7779E+05	0.6603
u237	1.1520E+00	7.6321E+00	8.7841E+00	7.6687E+02	1.2027E+01	1.0574E+03	0.7304
u238	3.4718E+05	5.8179E+05	9.2897E+05	8.1100E+07	9.2777E+05	8.1570E+07	1.0013
u239	2.3031E-01	4.1195E-01	6.4226E-01	5.6071E+01	6.4698E-01	5.6883E+01	0.9927
xel131m	6.5406E-02	1.0342E-01	1.6883E-01	1.4739E+01	1.6432E-01	1.4447E+01	1.0274
xel133	4.0965E+00	6.9020E+00	1.0998E+01	9.6018E+02	1.1017E+01	9.6858E+02	0.9984
xel133m	2.4525E-02	3.8395E-02	6.2920E-02	5.4930E+00	6.0992E-02	5.3625E+00	1.0316
xel135	1.2827E-01	1.4647E-01	2.7474E-01	2.3985E+01	2.3703E-01	2.0840E+01	1.1591
xel135m	1.5262E-03	2.2916E-03	3.8178E-03	3.3330E-01	3.6386E-03	3.1991E-01	1.0492
xel137	1.9687E-03	3.3552E-03	5.3239E-03	4.6478E-01	5.3594E-03	4.7120E-01	0.9934
xel138	6.5344E-03	1.1699E-02	1.8234E-02	1.5918E+00	1.8731E-02	1.6468E+00	0.9735
xel139	2.1014E-04	4.0285E-04	6.1299E-04	5.3515E-02	6.4669E-04	5.6857E-02	0.9479
xel140	4.4802E-05	9.3118E-05	1.3792E-04	1.2041E-02	1.4987E-04	1.3176E-02	0.9203
y90	2.9963E-02	1.0848E-01	1.3844E-01	1.2086E+01	1.7157E-01	1.5085E+01	0.8069
y91	1.2575E+01	3.1312E+01	4.3887E+01	3.8314E+03	5.0736E+01	4.4607E+03	0.8650
y91m	4.2096E-03	1.0225E-02	1.4434E-02	1.2601E+00	1.6574E-02	1.4572E+00	0.8709
y92	3.7543E-02	8.4970E-02	1.2251E-01	1.0696E+01	1.3739E-01	1.2079E+01	0.8917
y93	1.3404E-01	2.8131E-01	4.1535E-01	3.6261E+01	4.5358E-01	3.9879E+01	0.9157
y94	4.5847E-03	9.2416E-03	1.3826E-02	1.2071E+00	1.4876E-02	1.3079E+00	0.9294
y95	2.7975E-03	5.3961E-03	8.1936E-03	7.1531E-01	8.6700E-03	7.6228E-01	0.9450
y96	1.3655E-05	2.8094E-05	4.1749E-05	3.6448E-03	4.5217E-05	3.9755E-03	0.9233
zr95	2.5797E+01	5.0229E+01	7.6027E+01	6.6373E+03	8.0693E+01	7.0946E+03	0.9422
zr97	3.2069E-01	5.7382E-01	8.9451E-01	7.8092E+01	9.1857E-01	8.0762E+01	0.9738

<sup>a</sup>Core totals based on 87.302 MTHM in MOX core, 87.921 MTHM in LEU core.

Figure 11 graphically presents the MOX core/LEU core nuclide ratio data given in Table 6. The large difference that is apparent for Mo-92 is related to the quite different fission yields of this isotope for plutonium isotopes and U-235.

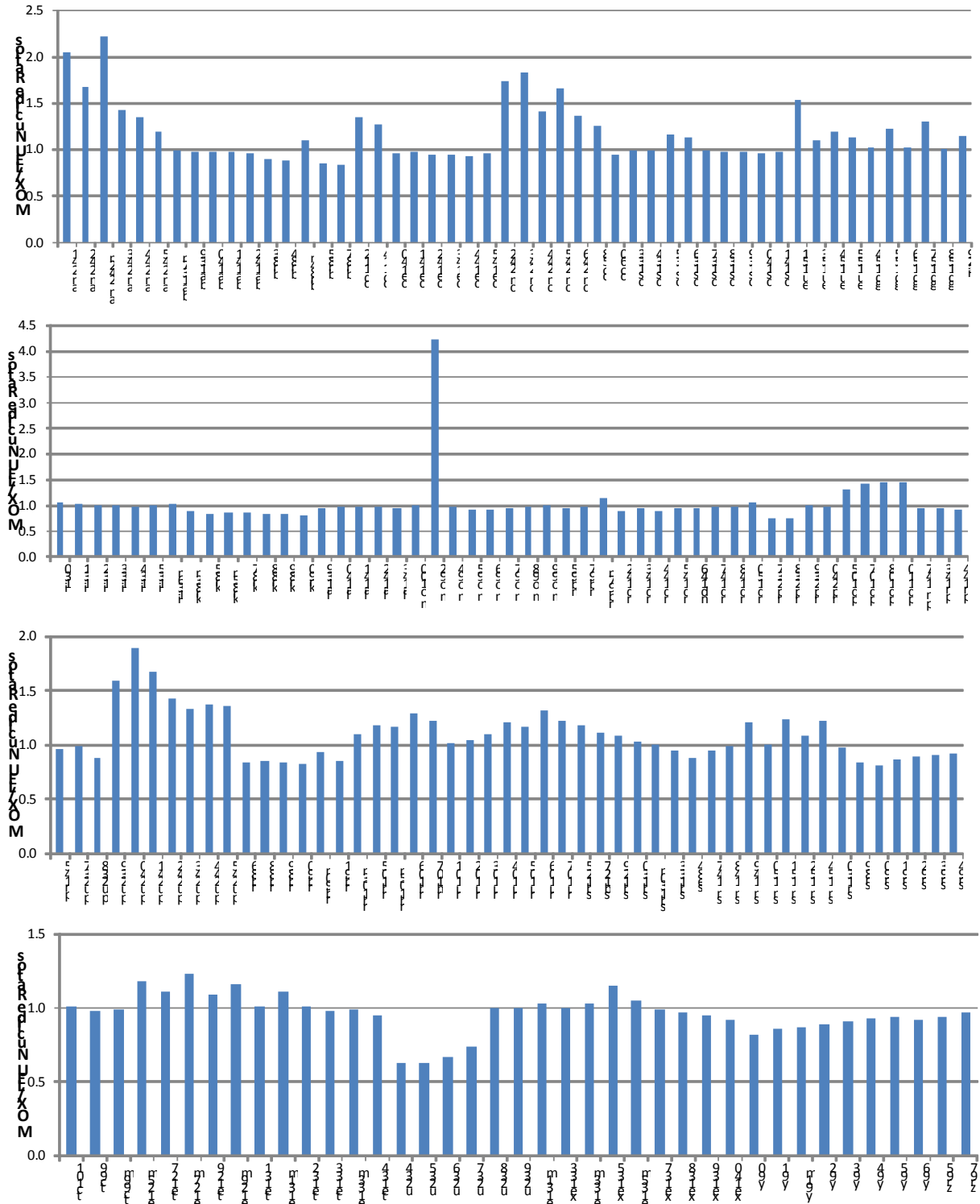


Figure 11. SQN MOX/LEU core-average nuclide ratios.

Table 7 compares the SQN MOX core /LEU core nuclide ratios for the current study to the corresponding values reported in Reference [3]. The percentage difference is calculated as

$$100*(1 - R_{i-current}/R_{i-EIS-0283}).$$

**Table 7. Comparison of SQN nuclide concentration ratios to Reference [3]**

Nuclide	Current study	EIS-0283 [3]		Percentage difference
	MOX/LEU ratios	Isotopes	MOX/LEU ratios	
am241	2.0199	Americium-241	2.06	1.9
am242	1.6450			
am242m	2.1830			
am243	1.3630			
am244	1.2901			
am245	1.1253			
ba137m	0.9784			
ba139	0.9791	Barium-139	0.97	-0.9
ba140	0.9757	Barium-140	0.98	0.4
ba141	0.9751			
ba142	0.9637			
br83	0.9003			
br84	0.8820			
br84m	1.0928			
br85	0.8606			
br87	0.8486			
cd112	1.3242			
cd114	1.2421			
ce140	0.9379			
ce141	0.9757	Cerium-141	0.98	0.4
ce142	0.9325			
ce143	0.9529	Cerium-143	0.95	-0.3
ce144	0.9245	Cerium-144	0.91	-1.6
ce145	0.9593			
cm242	1.6821	Curium-242	1.43	-17.6
cm243	1.7532			
cm244	1.3350	Curium-244	0.94	-42.0
cm245	1.5338			
cm246	1.2347			
co58	1.3232	Cobalt-58	0.86	-53.9
co60	0.9244	Cobalt-60	0.72	-28.4
cs133	0.9765			
cs134	0.9632	Cesium-134	0.85	-13.3
cs135	1.1409			
cs136	1.1180	Cesium-136	1.09	-2.6
cs137	0.9773	Cesium-137	0.91	-7.4
cs138	0.9801			
cs139	0.9766			
cs140	0.9565			
cs141	0.9738			
eu151	1.5402			
eu153	1.0817			
eu154	1.1627			

Table 7 (continued)

Nuclide	Current study	EIS-0283 [3]		Percentage difference
	MOX/LEU ratios	Isotopes	MOX/LEU ratios	
eu155	1.1084			
gd154	1.0288			
gd155	1.2224			
gd156	1.0173			
gd157	1.3042			
gd158	1.0139			
il29	1.1350			
il30	1.0258			
il31	1.0236	Iodine-131	1.03	0.6
il32	1.0164	Iodine-132	1.02	0.3
il33	0.9970	Iodine-133	1	0.3
il34	0.9877	Iodine-134	0.98	-0.8
il35	1.0005	Iodine-135	1	-0.1
il36m	1.0353			
kr83m	0.9004	Krypton-83m	0.89	-1.2
kr85	0.8199	Krypton-85	0.78	-5.1
kr85m	0.8604	Krypton-85m	0.86	0.0
kr87	0.8569	Krypton-87	0.85	-0.8
kr88	0.8493	Krypton-88	0.84	-1.1
kr89	0.8380			
kr90	0.8207			
la139	0.9444			
la140	0.9733	Lanthanum-140	0.97	-0.3
la141	0.9759	Lanthanum-141	0.97	-0.6
la142	0.9680	Lanthanum-142	0.97	0.2
la143	0.9533			
mo100	0.9761			
mo92	3.9709			
mo94	0.9359			
mo95	0.9047			
mo96	0.8758			
mo97	0.9439			
mo98	0.9619			
mo99	0.9936	Molybdenum-99	0.99	-0.4
nb95	0.9404	Niobium-95	0.94	0.0
nb97	0.9750			
nb97m	1.1449			
nd142	0.8681			
nd143	0.9445			
nd144	0.8675			
nd145	0.9249			
nd146	0.9247			
nd148	0.9642			
nd150	1.0429			
np237	0.7478			
np238	0.7383			
np239	0.9927	Neptunium-239	0.99	-0.3

Table 7 (continued)

Nuclide	Current study	EIS-0283 [3]		Percentage difference
	MOX/LEU ratios	Isotopes	MOX/LEU ratios	
np240	0.9709			
pd105	1.2880			
pd107	1.4052			
pd108	1.4363			
pd110	1.4190			
pm147	0.9580			
pr143	0.9528	Praseodymium-143	0.95	-0.3
pr144	0.9243			
pr145	0.9594			
pu237	0.9479			
pu238	0.8450	Plutonium-238	0.76	-11.2
pu239	1.5979	Plutonium-239	2.06	22.4
pu240	1.8855	Plutonium-240	2.2	14.3
pu241	1.6639	Plutonium-241	1.79	7.0
pu242	1.3883			
pu243	1.2947			
pu244	1.2898			
pu245	1.2778			
rb86	0.8240	Rubidium-86	0.77	-7.0
rb88	0.8529			
rb89	0.8489			
rb90	0.8276			
rb90m	0.9442			
rb91	0.8603			
rh103m	1.0966			
rh105	1.1726	Rhodium-105	1.19	1.5
rh105m	1.1626			
rh106	1.2713			
rh107	1.2175			
ru101	0.9946			
ru102	1.0304			
ru103	1.0965	Ruthenium-103	1.11	1.2
ru104	1.1878			
ru105	1.1624	Ruthenium-105	1.18	1.5
ru106	1.3036	Ruthenium-106	1.28	-1.8
ru107	1.2177			
sb125	1.1619			
sb127	1.1061	Antimony-127	1.15	3.8
sb129	1.0851	Antimony-129	1.07	-1.4
sb130	1.0252			
sb133	0.9438			
se84	0.8779			
sm147	0.9350			
sm148	0.9614			
sm149	1.2164			
sm150	0.9813			
sm151	1.2268			
sm152	1.0765			

Table 7 (continued)

Nuclide	Current study	EIS-0283 [3]		Percentage difference
	MOX/LEU ratios	Isotopes	MOX/LEU ratios	
sm154	1.1936			
sn130	0.9731			
sr89	0.8438	Strontium-89	0.83	-1.7
sr90	0.8084	Strontium-90	0.75	-7.8
sr91	0.8709	Strontium-91	0.86	-1.3
sr92	0.8920	Strontium-92	0.89	-0.2
sr93	0.9131			
sr94	0.9188			
tc101	1.0140			
tc99	0.9672			
tc99m	0.9936	Technetium-99m	0.99	-0.4
te125m	1.1693			
te127	1.1097	Tellurium-127	1.16	4.3
te127m	1.2293	Tellurium-127m	1.2	-2.4
te129	1.0852	Tellurium-129	1.08	-0.5
te129m	1.1520	Tellurium-129m	1.09	-5.7
te131	1.0115			
te131m	1.1108	Tellurium-131m	1.11	-0.1
te132	1.0124	Tellurium-132	1.01	-0.2
te133	0.9807			
te133m	0.9884			
te134	0.9557			
u234	0.6340			
u235	0.6410			
u236	0.6603			
u237	0.7304			
u238	1.0013			
u239	0.9927			
xe131m	1.0274	Xenon-131m	1.02	-0.7
xe133	0.9984	Xenon-133	1	0.2
xe133m	1.0316	Xenon-133m	1.01	-2.1
xe135	1.1591	Xenon-135	1.28	9.4
xe135m	1.0492	Xenon-135m	1.04	-0.9
xe137	0.9934			
xe138	0.9735	Xenon-138	0.96	-1.4
xe139	0.9479			
xe140	0.9203			
y91	0.8650	Yttrium-91	0.85	-1.8
y91m	0.8709			
y92	0.8917	Yttrium-92	0.89	-0.2
y93	0.9157	Yttrium-93	0.91	-0.6
y94	0.9294			
y95	0.9450			
y96	0.9233			
zr95	0.9422	Zirconium-95	0.94	-0.2
zr97	0.9738	Zirconium-97	0.98	0.6



Figure 12 presents the percentage differences of the MOX/LEU ratios of the current study to those of Reference [3] given in Table 7.

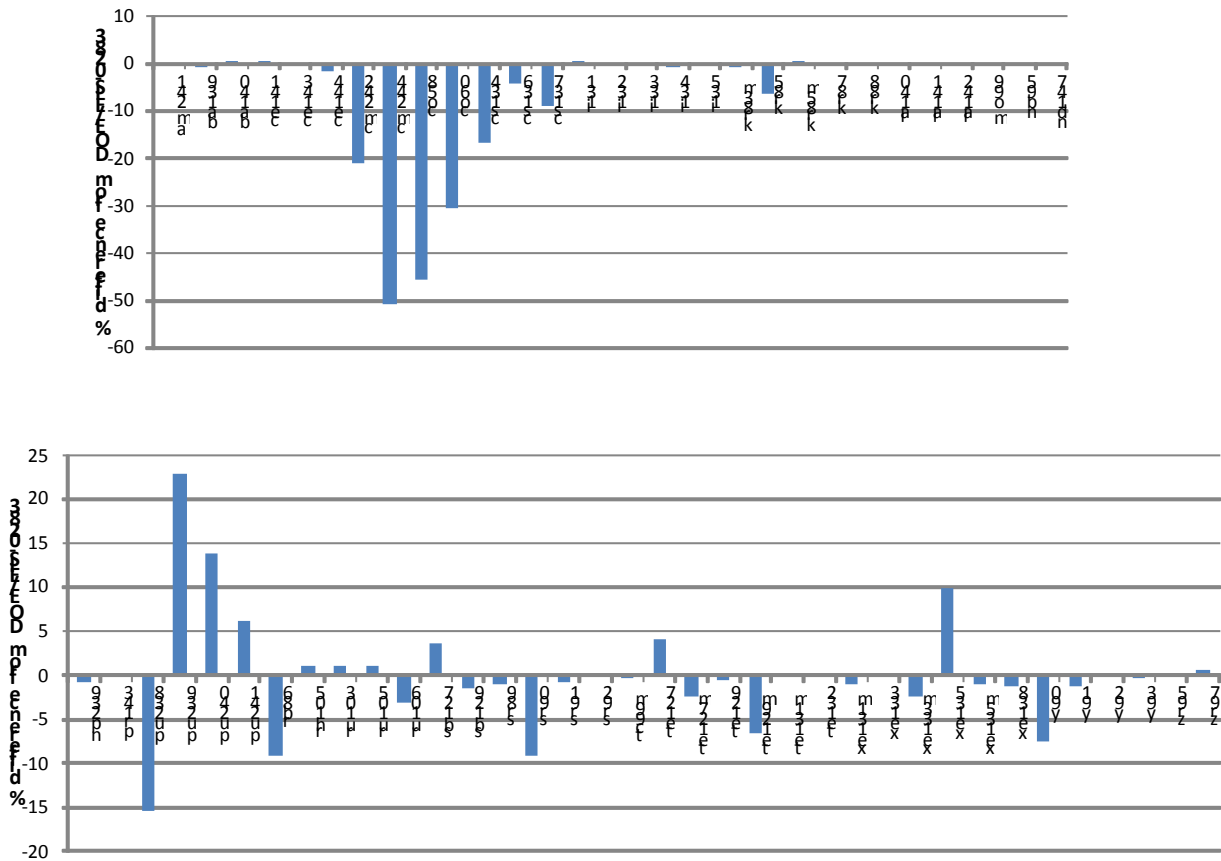


Figure 12. SQN % differences of MOX/LEU nuclide ratios from DOE/EIS-0283 values.

BFN

Table 8 presents a summary of the total mass of each of the considered nuclides in the BFN MOX and LEU cores as well as the ratios of the nuclide contents in the MOX core and the LEU core, respectively. The total isotopic masses were determined by multiplying the core-weighted-average isotopic mass per MTHM by the number of tonnes in each core (137.00 MTHM in MOX core, 138.67 MTHM in LEU core).

**Table 8. Summary of nuclide concentrations for the BFN equilibrium MOX and LEU cores**

Nuclide	MOX core				LEU core		Ratios
	Core-weighted-average components			Total mass <sup>a</sup>	Core-weighted-average of LEU assemblies	Total mass <sup>a</sup>	
	MOX assemblies	LEU assemblies	MOX+LEU assemblies				
	(g/MTHM)	(g/MTHM)	(g/MTHM)	(g)	(g/MTHM)	(g)	
am241	5.2603E+01	2.1374E+01	7.3977E+01	1.0135E+04	3.8521E+01	5.3417E+03	1.9204
am242	8.8702E-02	4.4986E-02	1.3369E-01	1.8315E+01	8.3278E-02	1.1548E+01	1.6053
am242m	7.0833E-01	2.7508E-01	9.8341E-01	1.3473E+02	4.9651E-01	6.8851E+01	1.9807
am243	7.9083E+01	5.0140E+01	1.2922E+02	1.7704E+04	1.0139E+02	1.4060E+04	1.2745
am244	3.2949E-03	2.2371E-03	5.5320E-03	7.5788E-01	4.5720E-03	6.3399E-01	1.2100
am245	9.3161E-07	7.4226E-07	1.6739E-06	2.2932E-04	1.5506E-06	2.1502E-04	1.0795
ba137m	8.1237E-05	1.0982E-04	1.9106E-04	2.6175E-02	2.0549E-04	2.8495E-02	0.9298
ba139	3.7098E-02	4.8494E-02	8.5592E-02	1.1726E+01	8.7061E-02	1.2073E+01	0.9831
ba140	7.9257E+00	1.0432E+01	1.8358E+01	2.5151E+03	1.8722E+01	2.5961E+03	0.9806
ba141	7.3904E-03	9.7545E-03	1.7145E-02	2.3488E+00	1.7499E-02	2.4266E+00	0.9797
ba142	3.9761E-03	5.3737E-03	9.3498E-03	1.2809E+00	9.6311E-03	1.3355E+00	0.9708
br83	2.1949E-03	3.4307E-03	5.6256E-03	7.7070E-01	6.1026E-03	8.4624E-01	0.9218
br84	7.9773E-04	1.3019E-03	2.0996E-03	2.8765E-01	2.3120E-03	3.2060E-01	0.9081
br84m	9.2891E-06	9.6909E-06	1.8980E-05	2.6003E-03	1.7601E-05	2.4407E-03	1.0783
br85	9.4292E-05	1.6220E-04	2.5650E-04	3.5140E-02	2.8727E-04	3.9835E-02	0.8929
br87	4.3931E-05	7.7779E-05	1.2171E-04	1.6674E-02	1.3767E-04	1.9091E-02	0.8841
cd112	8.7109E+00	6.2509E+00	1.4962E+01	2.0498E+03	1.1975E+01	1.6605E+03	1.2495
cd114	9.7000E+00	7.8800E+00	1.7580E+01	2.4085E+03	1.4999E+01	2.0799E+03	1.1721
ce140	4.8596E+02	7.2799E+02	1.2139E+03	1.6631E+05	1.3654E+03	1.8934E+05	0.8891
ce141	1.9163E+01	2.5230E+01	4.4394E+01	6.0819E+03	4.5290E+01	6.2803E+03	0.9802
ce142	4.4660E+02	6.7742E+02	1.1240E+03	1.5399E+05	1.2668E+03	1.7567E+05	0.8873
ce143	7.3392E-01	1.0159E+00	1.7498E+00	2.3972E+02	1.8183E+00	2.5214E+02	0.9624
ce144	1.2431E+02	1.8473E+02	3.0904E+02	4.2339E+04	3.3262E+02	4.6125E+04	0.9291
ce145	7.7893E-04	1.0627E-03	1.8416E-03	2.5230E-01	1.9037E-03	2.6399E-01	0.9674
cm242	1.3315E+01	6.5644E+00	1.9879E+01	2.7235E+03	1.2865E+01	1.7839E+03	1.5453
cm243	3.2214E-01	1.5148E-01	4.7362E-01	6.4886E+01	3.0569E-01	4.2390E+01	1.5494
cm244	2.7704E+01	1.8041E+01	4.5745E+01	6.2671E+03	3.7766E+01	5.2370E+03	1.2113
cm245	1.7595E+00	9.1022E-01	2.6697E+00	3.6575E+02	1.9531E+00	2.7084E+02	1.3669
cm246	2.1005E-01	1.2748E-01	3.3753E-01	4.6241E+01	2.8514E-01	3.9540E+01	1.1837
co58	4.4325E-18	2.8874E-18	7.3199E-18	1.0028E-15	5.7435E-18	7.9646E-16	1.2745
co60	3.9713E-15	5.8200E-15	9.7914E-15	1.3414E-12	1.0913E-14	1.5133E-12	0.8972
cs133	4.9492E+02	6.7293E+02	1.1678E+03	1.6000E+05	1.2530E+03	1.7375E+05	0.9320
cs134	4.5495E+01	6.4332E+01	1.0983E+02	1.5046E+04	1.2371E+02	1.7155E+04	0.8878
cs135	2.5167E+02	2.7667E+02	5.2834E+02	7.2382E+04	5.1289E+02	7.1122E+04	1.0301
cs136	2.5114E-01	2.7618E-01	5.2733E-01	7.2244E+01	5.2052E-01	7.2181E+01	1.0131
cs137	5.2924E+02	7.1719E+02	1.2464E+03	1.7076E+05	1.3418E+03	1.8607E+05	0.9289
cs138	1.5570E-02	2.0310E-02	3.5879E-02	4.9155E+00	3.6466E-02	5.0568E+00	0.9839
cs139	4.0318E-03	5.2978E-03	9.3296E-03	1.2782E+00	9.5080E-03	1.3185E+00	0.9812
cs140	3.7757E-04	5.1775E-04	8.9533E-04	1.2266E-01	9.2749E-04	1.2862E-01	0.9653
cs141	1.2068E-04	1.5934E-04	2.8001E-04	3.8362E-02	2.8602E-04	3.9662E-02	0.9790
eu151	1.3950E-02	9.9766E-03	2.3927E-02	3.2780E+00	1.7396E-02	2.4122E+00	1.3755
eu153	5.9751E+01	6.4757E+01	1.2451E+02	1.7058E+04	1.2222E+02	1.6949E+04	1.0187
eu154	1.2690E+01	1.2492E+01	2.5182E+01	3.4499E+03	2.3821E+01	3.3033E+03	1.0571
eu155	4.3371E+00	4.4466E+00	8.7837E+00	1.2034E+03	8.4894E+00	1.1772E+03	1.0347
gd154	7.1885E+01	1.3033E+02	2.0222E+02	2.7704E+04	2.2476E+02	3.1167E+04	0.8997

Table 8 (continued)

Nuclide	MOX core				LEU core		Ratios
	Core-weighted-average components			Total mass <sup>a</sup>	Core-weighted average of LEU assemblies	Total mass <sup>a</sup>	
	MOX assemblies	LEU assemblies	MOX+LEU assemblies				
	(g/MTHM)	(g/MTHM)	(g/MTHM)	(g)	(g/MTHM)	(g)	
gd155	1.7776E+00	-5.9321E-01	1.1844E+00	1.6226E+02	2.6641E+00	3.6943E+02	0.4446
gd156	1.3487E+03	2.4527E+03	3.8014E+03	5.2080E+05	4.2674E+03	5.9176E+05	0.8908
gd157	4.9087E-01	5.0099E-01	9.9186E-01	1.3588E+02	9.1033E-01	1.2624E+02	1.0896
gd158	1.5922E+03	2.8955E+03	4.4877E+03	6.1482E+05	5.0379E+03	6.9860E+05	0.8908
i129	8.8725E+01	8.7245E+01	1.7597E+02	2.4108E+04	1.6422E+02	2.2772E+04	1.0716
i130	3.8230E-03	4.3250E-03	8.1480E-03	1.1163E+00	8.3406E-03	1.1566E+00	0.9769
i131	2.9091E+00	3.4693E+00	6.3784E+00	8.7384E+02	6.2565E+00	8.6758E+02	1.0195
i132	5.0342E-02	6.0943E-02	1.1129E-01	1.5246E+01	1.0985E-01	1.5234E+01	1.0130
i133	6.2245E-01	7.8404E-01	1.4065E+00	1.9269E+02	1.4099E+00	1.9551E+02	0.9976
i134	2.9091E-02	3.7337E-02	6.6427E-02	9.1006E+00	6.7094E-02	9.3039E+00	0.9901
i135	1.9161E-01	2.3933E-01	4.3094E-01	5.9039E+01	4.3076E-01	5.9733E+01	1.0004
i136m	9.3852E-05	1.0895E-04	2.0280E-04	2.7784E-02	1.9694E-04	2.7310E-02	1.0298
kr83m	1.6622E-03	2.5984E-03	4.2606E-03	5.8370E-01	4.6236E-03	6.4116E-01	0.9215
kr85	8.9377E+00	1.8858E+01	2.7796E+01	3.8080E+03	3.4888E+01	4.8379E+03	0.7967
kr85m	8.4324E-03	1.4512E-02	2.2944E-02	3.1433E+00	2.5707E-02	3.5648E+00	0.8925
kr87	4.6699E-03	8.1054E-03	1.2775E-02	1.7502E+00	1.4352E-02	1.9902E+00	0.8902
kr88	1.3750E-02	2.4313E-02	3.8063E-02	5.2147E+00	4.3019E-02	5.9655E+00	0.8848
kr89	3.1275E-04	5.6934E-04	8.8208E-04	1.2085E-01	1.0061E-03	1.3951E-01	0.8767
kr90	5.3914E-05	1.0259E-04	1.5651E-04	2.1442E-02	1.8103E-04	2.5103E-02	0.8646
la139	4.9730E+02	7.3189E+02	1.2292E+03	1.6840E+05	1.3687E+03	1.8980E+05	0.8981
la140	1.0808E+00	1.4282E+00	2.5090E+00	3.4374E+02	2.5714E+00	3.5658E+02	0.9757
la141	9.5970E-02	1.2640E-01	2.2237E-01	3.0465E+01	2.2677E-01	3.1446E+01	0.9806
la142	3.5544E-02	4.7632E-02	8.3176E-02	1.1395E+01	8.5373E-02	1.1839E+01	0.9743
la143	5.2223E-03	7.2202E-03	1.2443E-02	1.7046E+00	1.2924E-02	1.7921E+00	0.9628
mo100	4.0885E+02	5.5658E+02	9.6543E+02	1.3226E+05	1.0428E+03	1.4460E+05	0.9258
mo92	2.1739E-12	3.7620E-13	2.5501E-12	3.4936E-10	7.6605E-13	1.0623E-10	3.3289
mo94	3.7144E-03	5.7401E-03	9.4545E-03	1.2953E+00	1.1334E-02	1.5716E+00	0.8342
mo95	2.4872E+02	4.0807E+02	6.5679E+02	8.9980E+04	7.6397E+02	1.0594E+05	0.8597
mo96	1.2387E+01	2.2699E+01	3.5086E+01	4.8067E+03	4.4642E+01	6.1905E+03	0.7859
mo97	3.2889E+02	4.8467E+02	8.1356E+02	1.1146E+05	9.0643E+02	1.2569E+05	0.8975
mo98	3.4919E+02	4.9209E+02	8.4128E+02	1.1526E+05	9.2170E+02	1.2781E+05	0.9127
mo99	1.3221E+00	1.6769E+00	2.9990E+00	4.1086E+02	3.0162E+00	4.1826E+02	0.9943
nb95	1.3273E+01	1.8828E+01	3.2101E+01	4.3978E+03	3.3691E+01	4.6719E+03	0.9528
nb97	2.0997E-02	2.7723E-02	4.8721E-02	6.6747E+00	4.9742E-02	6.8977E+00	0.9795
nb97m	4.2725E-07	4.0498E-07	8.3223E-07	1.1402E-04	7.3950E-07	1.0255E-04	1.1254
nd142	6.6077E+00	1.2061E+01	1.8668E+01	2.5576E+03	2.3814E+01	3.3023E+03	0.7839
nd143	3.2077E+02	4.7401E+02	7.9478E+02	1.0889E+05	8.6990E+02	1.2063E+05	0.9136
nd144	3.3804E+02	6.1566E+02	9.5370E+02	1.3066E+05	1.1788E+03	1.6347E+05	0.8090
nd145	2.6227E+02	4.0556E+02	6.6783E+02	9.1493E+04	7.5358E+02	1.0450E+05	0.8862
nd146	2.7112E+02	4.2043E+02	6.9156E+02	9.4743E+04	7.9115E+02	1.0971E+05	0.8741
nd147	2.7168E+00	3.5162E+00	6.2330E+00	8.5392E+02	6.3202E+00	8.7642E+02	0.9862
nd148	1.5755E+02	2.2034E+02	3.7789E+02	5.1772E+04	4.1269E+02	5.7228E+04	0.9157
nd150	8.7902E+01	1.0301E+02	1.9091E+02	2.6155E+04	1.9365E+02	2.6854E+04	0.9858
np237	8.8554E+01	2.3836E+02	3.2691E+02	4.4787E+04	4.4805E+02	6.2131E+04	0.7296
np238	1.8909E-01	5.5154E-01	7.4062E-01	1.0147E+02	1.0576E+00	1.4665E+02	0.7003

Table 8 (continued)

Nuclide	MOX core				LEU core		Ratios
	Core-weighted-average components			Total mass <sup>a</sup>	Core-weighted average of LEU assemblies	Total mass <sup>a</sup>	
	MOX assemblies	LEU assemblies	MOX+LEU assemblies				
	(g/MTHM)	(g/MTHM)	(g/MTHM)	(g)	(g/MTHM)	(g)	
np239	2.6209E+01	3.2915E+01	5.9124E+01	8.1000E+03	6.0452E+01	8.3828E+03	0.9780
np240	2.3042E-04	2.9642E-04	5.2684E-04	7.2177E-02	5.5588E-04	7.7084E-02	0.9478
pd105	2.7603E+02	2.1126E+02	4.8729E+02	6.6759E+04	4.0095E+02	5.5600E+04	1.2153
pd107	1.7339E+02	1.1079E+02	2.8418E+02	3.8932E+04	2.1244E+02	2.9460E+04	1.3377
pd108	1.1830E+02	7.2303E+01	1.9060E+02	2.6113E+04	1.3917E+02	1.9298E+04	1.3696
pd110	3.8356E+01	2.3991E+01	6.2347E+01	8.5416E+03	4.6206E+01	6.4073E+03	1.3493
pm147	7.9916E+01	1.1105E+02	1.9096E+02	2.6162E+04	2.0119E+02	2.7899E+04	0.9492
pr143	7.1210E+00	9.8881E+00	1.7009E+01	2.3302E+03	1.7663E+01	2.4493E+03	0.9630
pr144	5.2607E-03	7.8192E-03	1.3080E-02	1.7919E+00	1.4082E-02	1.9528E+00	0.9288
pr145	9.3006E-02	1.2683E-01	2.1984E-01	3.0118E+01	2.2723E-01	3.1510E+01	0.9675
pu237	1.2811E-04	1.6906E-04	2.9717E-04	4.0712E-02	3.3431E-04	4.6359E-02	0.8889
pu238	4.6780E+01	8.2018E+01	1.2880E+02	1.7645E+04	1.6036E+02	2.2238E+04	0.8032
pu239	4.6387E+03	2.7180E+03	7.3567E+03	1.0079E+06	4.8744E+03	6.7593E+05	1.5093
pu240	2.6745E+03	1.1439E+03	3.8184E+03	5.2312E+05	2.1364E+03	2.9625E+05	1.7873
pu241	1.0848E+03	5.6609E+02	1.6509E+03	2.2617E+05	1.0537E+03	1.4612E+05	1.5667
pu242	3.9196E+02	2.3681E+02	6.2877E+02	8.6142E+04	4.6668E+02	6.4714E+04	1.3473
pu243	7.1116E-02	4.8849E-02	1.1996E-01	1.6435E+01	9.6600E-02	1.3396E+01	1.2419
pu244	1.9123E-02	1.2844E-02	3.1967E-02	4.3795E+00	2.6812E-02	3.7180E+00	1.1923
pu245	8.4290E-07	5.9575E-07	1.4387E-06	1.9710E-04	1.2594E-06	1.7465E-04	1.1423
rb86	6.1961E-03	1.2866E-02	1.9062E-02	2.6115E+00	2.4375E-02	3.3800E+00	0.7821
rb88	1.4705E-03	2.5784E-03	4.0488E-03	5.5469E-01	4.5638E-03	6.3287E-01	0.8872
rb89	1.6541E-03	2.9297E-03	4.5838E-03	6.2798E-01	5.1827E-03	7.1868E-01	0.8845
rb90	2.8049E-04	5.2445E-04	8.0494E-04	1.1028E-01	9.2601E-04	1.2841E-01	0.8693
rb90m	1.1089E-04	1.5719E-04	2.6809E-04	3.6728E-02	2.8058E-04	3.8907E-02	0.9555
rb91	1.3604E-04	2.3405E-04	3.7010E-04	5.0703E-02	4.1464E-04	5.7498E-02	0.8926
rh103m	1.9585E-02	2.0234E-02	3.9819E-02	5.4551E+00	3.6850E-02	5.1099E+00	1.0806
rh105	5.4787E-01	4.9235E-01	1.0402E+00	1.4251E+02	9.0504E-01	1.2550E+02	1.1494
rh105m	5.5626E-05	5.0555E-05	1.0618E-04	1.4547E-02	9.2953E-05	1.2890E-02	1.1423
rh106	9.7651E-05	7.4154E-05	1.7181E-04	2.3537E-02	1.3884E-04	1.9252E-02	1.2375
rh107	3.6697E-03	3.0121E-03	6.6818E-03	9.1540E-01	5.5848E-03	7.7445E-01	1.1964
ru101	3.5366E+02	4.6096E+02	8.1462E+02	1.1160E+05	8.6287E+02	1.1965E+05	0.9441
ru102	3.8465E+02	4.6312E+02	8.4777E+02	1.1614E+05	8.7205E+02	1.2093E+05	0.9722
ru103	1.9958E+01	2.0618E+01	4.0576E+01	5.5589E+03	3.7552E+01	5.2073E+03	1.0805
ru104	3.4229E+02	3.0510E+02	6.4738E+02	8.8692E+04	5.7804E+02	8.0157E+04	1.1200
ru105	7.3526E-02	6.6824E-02	1.4035E-01	1.9228E+01	1.2288E-01	1.7039E+01	1.1422
ru106	9.8207E+01	7.2287E+01	1.7049E+02	2.3358E+04	1.3521E+02	1.8750E+04	1.2610
ru107	6.2471E-04	5.1258E-04	1.1373E-03	1.5581E-01	9.5049E-04	1.3180E-01	1.1965
sb125	4.9491E+00	4.6088E+00	9.5579E+00	1.3094E+03	8.5818E+00	1.1900E+03	1.1137
sb127	1.4642E-01	1.4874E-01	2.9516E-01	4.0437E+01	2.7085E-01	3.7559E+01	1.0898
sb129	2.0749E-02	2.1904E-02	4.2654E-02	5.8436E+00	3.9805E-02	5.5197E+00	1.0716
sb130	2.8105E-03	3.3397E-03	6.1502E-03	8.4258E-01	6.0265E-03	8.3570E-01	1.0205
sb130m	4.5466E-04	5.6433E-04	1.0190E-03	1.3960E-01	1.0156E-03	1.4083E-01	1.0033
sb133	3.4010E-04	4.7835E-04	8.1845E-04	1.1213E-01	8.5673E-04	1.1880E-01	0.9553
se84	7.4863E-05	1.2338E-04	1.9824E-04	2.7159E-02	2.1900E-04	3.0369E-02	0.9052
sm147	3.4525E+01	5.2892E+01	8.7418E+01	1.1976E+04	9.9655E+01	1.3819E+04	0.8772

Table 8 (continued)

Nuclide	MOX core				LEU core		Ratios
	Core-weighted-average components			Total mass <sup>a</sup>	Core-weighted average of LEU assemblies	Total mass <sup>a</sup>	
	MOX assemblies	LEU assemblies	MOX+LEU assemblies				
	(g/MTHM)	(g/MTHM)	(g/MTHM)	(g)	(g/MTHM)	(g)	
sm148	5.1403E+01	7.5439E+01	1.2684E+02	1.7377E+04	1.4554E+02	2.0181E+04	0.8716
sm149	1.2403E+00	1.2190E+00	2.4593E+00	3.3693E+02	2.1511E+00	2.9830E+02	1.1433
sm150	1.2462E+02	1.6601E+02	2.9063E+02	3.9816E+04	3.1161E+02	4.3211E+04	0.9327
sm151	6.9904E+00	6.5826E+00	1.3573E+01	1.8595E+03	1.1825E+01	1.6398E+03	1.1478
sm152	5.7794E+01	6.3214E+01	1.2101E+02	1.6578E+04	1.1698E+02	1.6221E+04	1.0345
sm154	2.2657E+01	1.9829E+01	4.2486E+01	5.8206E+03	3.7643E+01	5.2199E+03	1.1287
sn130	1.5453E-04	2.0440E-04	3.5893E-04	4.9173E-02	3.6689E-04	5.0876E-02	0.9783
sr89	8.1448E+00	1.4559E+01	2.2704E+01	3.1104E+03	2.5787E+01	3.5759E+03	0.8804
sr90	1.5161E+02	3.3491E+02	4.8652E+02	6.6653E+04	6.2138E+02	8.6167E+04	0.7830
sr91	8.8731E-02	1.4877E-01	2.3750E-01	3.2537E+01	2.6383E-01	3.6585E+01	0.9002
sr92	2.8004E-02	4.4634E-02	7.2638E-02	9.9514E+00	7.9322E-02	1.1000E+01	0.9157
sr93	1.5326E-03	2.3239E-03	3.8565E-03	5.2834E-01	4.1397E-03	5.7405E-01	0.9316
sr94	2.6147E-04	3.9117E-04	6.5264E-04	8.9411E-02	6.9723E-04	9.6684E-02	0.9360
tc101	4.6757E-03	5.6775E-03	1.0353E-02	1.4184E+00	1.0239E-02	1.4199E+00	1.0111
tc99	3.4380E+02	4.7711E+02	8.2091E+02	1.1246E+05	8.8889E+02	1.2326E+05	0.9235
tc99m	1.0672E-01	1.3546E-01	2.4218E-01	3.3179E+01	2.4360E-01	3.3780E+01	0.9942
te125m	6.2694E-02	5.7838E-02	1.2053E-01	1.6513E+01	1.0807E-01	1.4987E+01	1.1153
te127	1.4559E-02	1.4733E-02	2.9291E-02	4.0129E+00	2.6793E-02	3.7154E+00	1.0932
te127m	1.2261E-01	1.0901E-01	2.3163E-01	3.1733E+01	1.9585E-01	2.7158E+01	1.1827
te129	5.4782E-03	5.7818E-03	1.1260E-02	1.5426E+00	1.0507E-02	1.4570E+00	1.0716
te129m	1.7969E-03	1.6971E-03	3.4940E-03	4.7868E-01	3.1379E-03	4.3513E-01	1.1135
te131	5.4951E-03	6.7128E-03	1.2208E-02	1.6725E+00	1.2097E-02	1.6775E+00	1.0092
te131m	7.2840E-02	7.3480E-02	1.4632E-01	2.0046E+01	1.3375E-01	1.8547E+01	1.0940
te132	1.6312E+00	1.9907E+00	3.6219E+00	4.9620E+02	3.5875E+00	4.9748E+02	1.0096
te133	3.4882E-03	4.5369E-03	8.0251E-03	1.0994E+00	8.1496E-03	1.1301E+00	0.9847
te133m	1.1405E-02	1.4634E-02	2.6039E-02	3.5674E+00	2.6286E-02	3.6450E+00	0.9906
te134	1.7267E-02	2.3715E-02	4.0982E-02	5.6145E+00	4.2472E-02	5.8896E+00	0.9649
u234	3.7476E+01	1.3726E+02	1.7474E+02	2.3940E+04	2.4126E+02	3.3455E+04	0.7243
u235	3.2192E+03	8.5584E+03	1.1778E+04	1.6135E+06	1.4540E+04	2.0163E+06	0.8100
u236	6.8441E+02	2.8704E+03	3.5548E+03	4.8700E+05	5.2598E+03	7.2937E+05	0.6758
u237	1.4114E+00	3.9391E+00	5.3505E+00	7.3302E+02	7.2892E+00	1.0108E+03	0.7340
u238	4.1715E+05	5.1769E+05	9.3484E+05	1.2807E+08	9.3199E+05	1.2924E+08	1.0031
u239	1.8153E-01	2.2799E-01	4.0953E-01	5.6105E+01	4.1871E-01	5.8063E+01	0.9781
xe131m	5.6262E-02	6.6714E-02	1.2298E-01	1.6848E+01	1.2084E-01	1.6756E+01	1.0177
xe133	3.6495E+00	4.5764E+00	8.2259E+00	1.1270E+03	8.2472E+00	1.1436E+03	0.9974
xe133m	2.1242E-02	2.4825E-02	4.6067E-02	6.3112E+00	4.5179E-02	6.2650E+00	1.0196
xe135	9.9267E-02	1.0289E-01	2.0216E-01	2.7696E+01	1.8139E-01	2.5153E+01	1.1145
xe135m	1.2870E-03	1.4602E-03	2.7472E-03	3.7637E-01	2.6402E-03	3.6612E-01	1.0405
xe137	1.7504E-03	2.2190E-03	3.9694E-03	5.4380E-01	3.9905E-03	5.5337E-01	0.9947
xe138	5.9330E-03	7.8416E-03	1.3775E-02	1.8871E+00	1.4073E-02	1.9515E+00	0.9788
xe139	1.9658E-04	2.7446E-04	4.7103E-04	6.4531E-02	4.9129E-04	6.8127E-02	0.9588
xe140	4.3420E-05	6.4439E-05	1.0786E-04	1.4777E-02	1.1510E-04	1.5962E-02	0.9371
y90	3.9511E-02	8.7707E-02	1.2722E-01	1.7429E+01	1.6291E-01	2.2591E+01	0.7809
y91	1.3158E+01	2.2292E+01	3.5449E+01	4.8565E+03	3.9577E+01	5.4881E+03	0.8957
y91m	4.3713E-03	7.3277E-03	1.1699E-02	1.6028E+00	1.2994E-02	1.8019E+00	0.9003

Table 8 (continued)

Nuclide	MOX core			LEU core			Ratios
	Core-weighted-average components			Total mass <sup>a</sup>	Core-weighted average of LEU assemblies	Total mass <sup>a</sup>	
	MOX assemblies	LEU assemblies	MOX+LEU assemblies				
	(g/MTHM)	(g/MTHM)	(g/MTHM)	(g)	(g/MTHM)	(g)	
y92	3.7689E-02	6.0075E-02	9.7763E-02	1.3394E+01	1.0678E-01	1.4808E+01	0.9155
y93	1.2995E-01	1.9589E-01	3.2585E-01	4.4641E+01	3.4903E-01	4.8400E+01	0.9336
y94	4.3668E-03	6.3801E-03	1.0747E-02	1.4723E+00	1.1385E-02	1.5788E+00	0.9439
y95	2.6141E-03	3.6862E-03	6.3003E-03	8.6314E-01	6.5889E-03	9.1368E-01	0.9562
y96	1.3131E-05	1.9392E-05	3.2523E-05	4.4557E-03	3.4621E-05	4.8009E-03	0.9394
zr95	2.4245E+01	3.4297E+01	5.8542E+01	8.0203E+03	6.1353E+01	8.5078E+03	0.9542
zr97	2.9000E-01	3.8380E-01	6.7380E-01	9.2310E+01	6.8861E-01	9.5489E+01	0.9759

<sup>a</sup>Core totals based on 137.00 MTHM in MOX core, 138.67 MTHM in LEU core.

Table 9 compares the BFN MOX core/LEU core nuclide ratios for the current study to those reported in Reference [3]. The percentage difference is calculated as

$$100*(1 - R_{i-current}/R_{i-EIS-0283}).$$

Table 9. Comparison of BFN nuclide concentration ratios to Reference [3]

Nuclide	Current study	EIS-0283 [3]		Percentage difference
	MOX/LEU ratios	Nuclide	MOX/LEU ratios	
am241	1.9204	Americium-241	2.06	6.8
am242	1.6053			
am242m	1.9807			
am243	1.2745			
am244	1.2100			
am245	1.0795			
ba137m	0.9298			
ba139	0.9831	Barium-139	0.97	-1.4
ba140	0.9806	Barium-140	0.98	-0.1
ba141	0.9797	Praeseodymium-222		
ba142	0.9708			
br83	0.9218			
br84	0.9081			
br84m	1.0783			
br85	0.8929			
br87	0.8841			
cd112	1.2495			
cd114	1.1721			
ce140	0.8891			
ce141	0.9802	Cerium-141	0.98	0.0
ce142	0.8873			
ce143	0.9624	Cerium-143	0.95	-1.3
ce144	0.9291	Cerium-144	0.91	-2.1
ce145	0.9674			
cm242	1.5453	Curium-242	1.43	-8.1
cm243	1.5494			
cm244	1.2113	Curium-244	0.94	-28.9

Table 9 (continued)

Nuclide	Current study	EIS-0283 [3]		Percentage difference
	MOX/LEU ratios	Nuclide	MOX/LEU ratios	
cm245	1.3669			
cm246	1.1837			
co58	1.2745	Cobalt-58	0.86	-48.2
co60	0.8972	Cobalt-60	0.72	-24.6
cs133	0.9320			
cs134	0.8878	Cesium-134	0.85	-4.4
cs135	1.0301			
cs136	1.0131	Cesium-136	1.09	7.1
cs137	0.9289	Cesium-137	0.91	-2.1
cs138	0.9839			
cs139	0.9812			
cs140	0.9653			
cs141	0.9790			
eu151	1.3755			
eu153	1.0187			
eu154	1.0571			
eu155	1.0347			
gd154	0.8997			
gd155	0.4446			
gd156	0.8908			
gd157	1.0896			
gd158	0.8908			
i129	1.0716			
i130	0.9769			
i131	1.0195	Iodine-131	1.03	1.0
i132	1.0130	Iodine-132	1.02	0.7
i133	0.9976	Iodine-133	1	0.2
i134	0.9901	Iodine-134	0.98	-1.0
i135	1.0004	Iodine-135	1	0.0
i136m	1.0298			
kr83m	0.9215	Krypton-83m	0.89	-3.5
kr85	0.7967	Krypton-85	0.78	-2.1
kr85m	0.8925	Krypton-85m	0.86	-3.8
kr87	0.8902	Krypton-87	0.85	-4.7
kr88	0.8848	Krypton 88	0.84	-5.3
kr89	0.8767			
kr90	0.8646			
la139	0.8981			
la140	0.9757	Lanthanum-140	0.97	-0.6
la141	0.9806	Lanthanum-141	0.97	-1.1
la142	0.9743	Lanthanum-142	0.97	-0.4
la143	0.9628			
mo100	0.9258			
mo92	3.3289			
mo94	0.8342			
mo95	0.8597			
mo96	0.7859			
mo97	0.8975			
mo98	0.9127			



Table 9 (continued)

Nuclide	Current study	EIS-0283 [3]		Percentage difference
	MOX/LEU ratios	Nuclide	MOX/LEU ratios	
mo99	0.9943	Molybdenum-99	0.99	-0.4
nb95	0.9528	Niobium-95	0.94	-1.4
nb97	0.9795			
nb97m	1.1254			
nd142	0.7839			
nd143	0.9136			
nd144	0.8090			
nd145	0.8862			
nd146	0.8741			
nd147	0.9862	Neodymium-147	0.98	-0.6
nd148	0.9157			
nd150	0.9858			
np237	0.7296			
np238	0.7003			
np239	0.9780	Neptunium-239	0.99	1.2
np240	0.9478			
pd105	1.2153			
pd107	1.3377			
pd108	1.3696			
pd110	1.3493			
pm147	0.9492			
pr143	0.9630	Praseodymium-143	0.95	-1.4
pr144	0.9288			
pr145	0.9675			
pu237	0.8889			
pu238	0.8032	Plutonium-238	0.76	-5.7
pu239	1.5093	Plutonium-239	2.06	26.7
pu240	1.7873	Plutonium-240	2.2	18.8
pu241	1.5667	Plutonium-241	1.79	12.5
pu242	1.3473			
pu243	1.2419			
pu244	1.1923			
pu245	1.1423			
rb86	0.7821	Rubidium-86	0.77	-1.6
rb88	0.8872			
rb89	0.8845			
rb90	0.8693			
rb90m	0.9555			
rb91	0.8926			
rh103m	1.0806			
rh105	1.1494	Rhodium-105	1.19	3.4
rh105m	1.1423			
rh106	1.2375			
rh107	1.1964			
ru101	0.9441			
ru102	0.9722			
ru103	1.0805	Ruthenium-103	1.11	2.7
ru104	1.1200			
ru105	1.1422	Ruthenium-105	1.18	3.2

Table 9 (continued)

Nuclide	Current study	EIS-0283 [3]		Percentage difference
	MOX/LEU ratios	Nuclide	MOX/LEU ratios	
ru106	1.2610	Ruthenium-106	1.28	1.5
ru107	1.1965			
sb125	1.1137			
sb127	1.0898	Antimony-127	1.15	5.2
sb129	1.0716	Antimony-129	1.07	-0.1
sb130	1.0205			
sb130m	1.0033			
sb133	0.9553			
se84	0.9052			
sm147	0.8772			
sm148	0.8716			
sm149	1.1433			
sm150	0.9327			
sm151	1.1478			
sm152	1.0345			
sm154	1.1287			
sn130	0.9783			
sr89	0.8804	Strontium-89	0.83	-6.1
sr90	0.7830	Strontium-90	0.75	-4.4
sr91	0.9002	Strontium-91	0.86	-4.7
sr92	0.9157	Strontium-92	0.89	-2.9
sr93	0.9316			
sr94	0.9360			
tc101	1.0111			
tc99	0.9235			
tc99m	0.9942	Technetium-99m	0.99	-0.4
tel125m	1.1153			
tel127	1.0932	Tellurium-127	1.16	5.8
tel127m	1.1827	Tellurium-127m	1.2	1.4
tel129	1.0716	Tellurium-129	1.08	0.8
tel129m	1.1135	Tellurium-129m	1.09	-2.2
tel131	1.0092			
tel131m	1.0940	Tellurium-131m	1.11	1.4
tel132	1.0096	Tellurium-132	1.01	0.0
tel133	0.9847			
tel133m	0.9906			
tel134	0.9649			
u234	0.7243			
u235	0.8100			
u236	0.6758			
u237	0.7340			
u238	1.0031			
u239	0.9781			
xe131m	1.0177	Xenon-131m	1.02	0.2
xe133	0.9974	Xenon-133	1	0.3
xe133m	1.0196	Xenon-133m	1.01	-1.0
xe135	1.1145	Xenon-135	1.28	12.9
xe135m	1.0405	Xenon-135m	1.04	-0.1
xe137	0.9947			

**Table 9 (continued)**

Nuclide	Current study	EIS-0283 [3]		Percentage difference
	MOX/LEU ratios	Nuclide	MOX/LEU ratios	
xe138	0.9788	Xenon-138	0.96	-2.0
xe139	0.9588			
xe140	0.9371			
y90	0.7809	Yttrium-90	0.76	-2.7
y91	0.8957	Yttrium-91	0.85	-5.4
y91m	0.9003			
y92	0.9155	Yttrium-92	0.89	-2.9
y93	0.9336	Yttrium-93	0.91	-2.6
y94	0.9439			
y95	0.9562			
y96	0.9394			
zr95	0.9542	Zirconium-95	0.94	-1.5
zr97	0.9759	Zirconium-97	0.98	0.4

Figure 13 graphically presents the MOX core/LEU core nuclide ratio data given in Table 9. The large difference that is apparent for Mo-92 is related to the quite different fission yields of this isotope for plutonium isotopes and U-235.

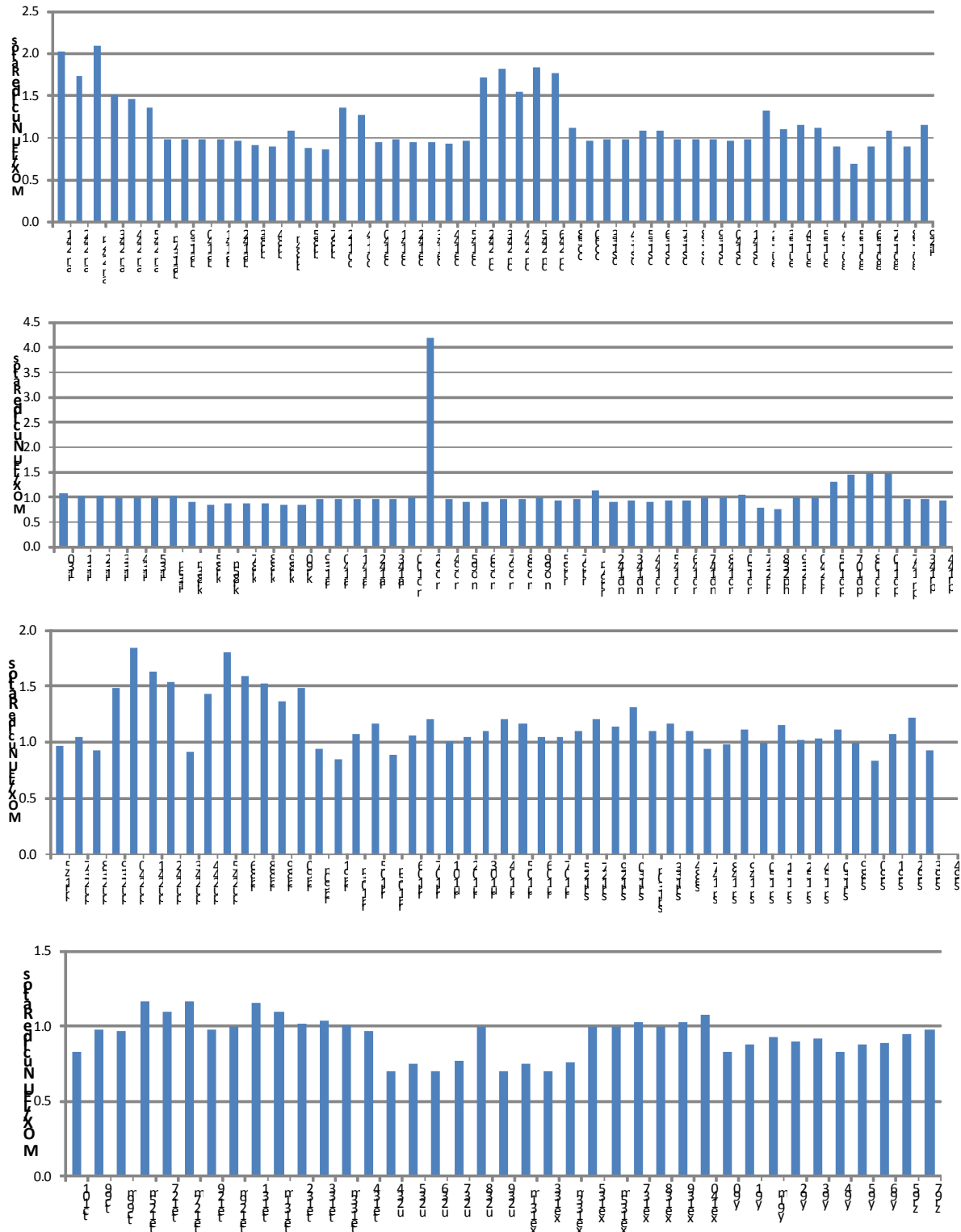


Figure 13. BFN MOX/LEU core-average nuclide ratios.

Figure 14 presents the percentage differences of the MOX/LEU ratios of the current study to the PWR values of Reference [3] given in Table 9.

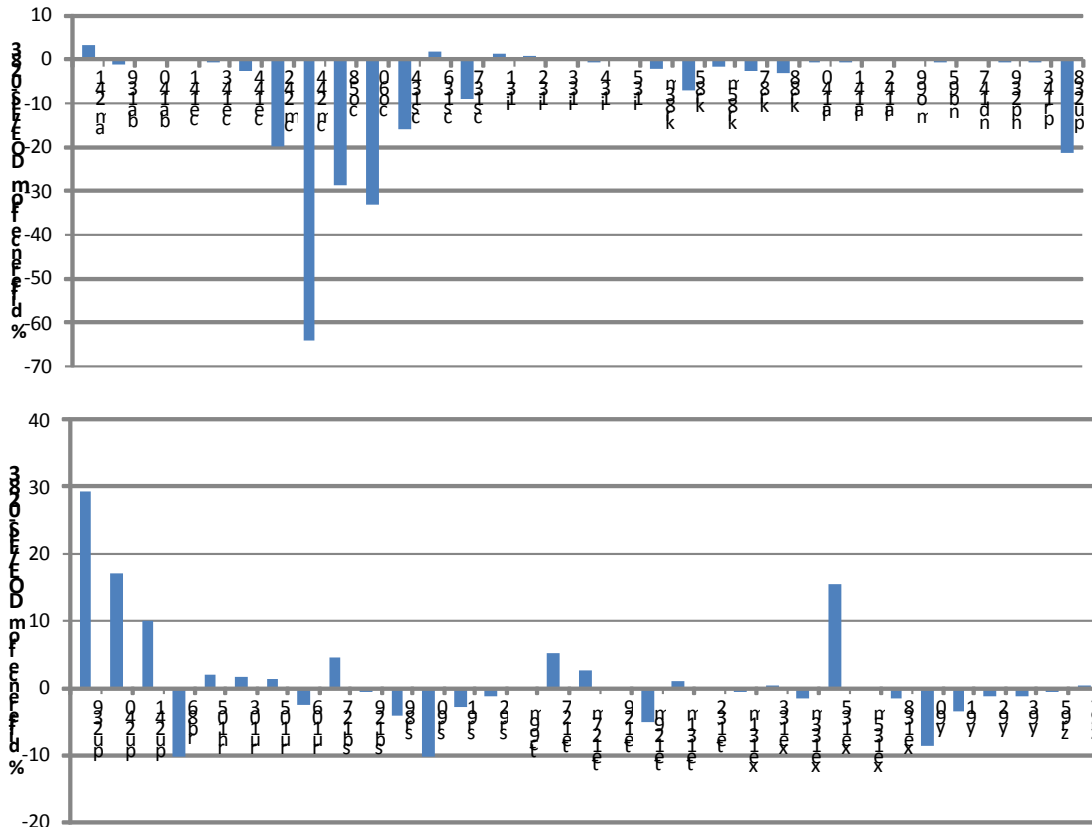


Figure 14. BFN % differences of MOX/LEU nuclide ratios from DOE/EIS-0283 PWR values.

## 9.4 Radioactivity Sources

### SQN

Table 10 presents a comparison of the contribution of each nuclide to the radioactivity source in the SQN equilibrium MOX and LEU cores. The core-average activity in curies/MTHM as output by SCALE/ORIGEN has been multiplied by the number of metric tonnes of heavy metal in each core to determine the core total activity. There are 87.921 MTHM in the LEU core and 87.302 MTHM in the MOX core.

**Table 10. Comparison of SQN radioactivity sources**

Nuclide	MOX core		LEU core	
	Curies/MTHM	Total curies	Curies/MTHM	Total curies
am241	3.20E+02	2.79E+04	1.53E+02	1.35E+04
am242	1.63E+05	1.42E+07	9.39E+04	8.25E+06
am242m	1.59E+01	1.39E+03	7.05E+00	6.20E+02
am243	4.13E+01	3.60E+03	2.59E+01	2.28E+03
am244	1.62E+04	1.42E+06	1.05E+04	9.20E+05
am245	2.64E+01	2.30E+03	1.78E+01	1.57E+03
ba137m	1.24E+05	1.08E+07	1.20E+05	1.05E+07
ba139	1.85E+06	1.61E+08	1.89E+06	1.66E+08
ba140	1.77E+06	1.54E+08	1.82E+06	1.60E+08
ba141	1.65E+06	1.44E+08	1.70E+06	1.50E+08
ba142	1.53E+06	1.33E+08	1.60E+06	1.40E+08
br83	1.10E+05	9.58E+06	1.24E+05	1.09E+07
br84	1.80E+05	1.57E+07	2.09E+05	1.84E+07
br84m	1.02E+04	8.89E+05	9.24E+03	8.12E+05
br85	2.33E+05	2.04E+07	2.78E+05	2.45E+07
br87	3.35E+05	2.92E+07	4.06E+05	3.57E+07
cd112	0.00E+00	0.00E+00	0.00E+00	0.00E+00
cd114	0.00E+00	0.00E+00	0.00E+00	0.00E+00
ce140	0.00E+00	0.00E+00	0.00E+00	0.00E+00
ce141	1.66E+06	1.45E+08	1.71E+06	1.50E+08
ce142	0.00E+00	0.00E+00	0.00E+00	0.00E+00
ce143	1.50E+06	1.31E+08	1.59E+06	1.40E+08
ce144	1.25E+06	1.09E+08	1.35E+06	1.19E+08
ce145	1.04E+06	9.04E+07	1.09E+06	9.55E+07
cm242	9.35E+04	8.16E+06	5.15E+04	4.53E+06
cm243	3.99E+01	3.48E+03	2.02E+01	1.78E+03
cm244	7.38E+03	6.44E+05	4.23E+03	3.72E+05
cm245	1.32E+00	1.15E+02	6.20E-01	5.45E+01
cm246	2.80E-01	2.45E+01	1.18E-01	1.04E+01
co58	2.93E-13	2.56E-11	2.21E-13	1.95E-11
co60	1.34E-11	1.17E-09	1.36E-11	1.19E-09
cs133	0.00E+00	0.00E+00	0.00E+00	0.00E+00
cs134	2.27E+05	1.98E+07	2.11E+05	1.86E+07
cs135	6.82E-01	5.95E+01	5.69E-01	5.01E+01
cs136	6.22E+04	5.43E+06	5.23E+04	4.60E+06
cs137	1.30E+05	1.14E+07	1.26E+05	1.11E+07
cs138	1.94E+06	1.69E+08	1.99E+06	1.75E+08
cs139	1.80E+06	1.57E+08	1.85E+06	1.63E+08
cs140	1.47E+06	1.29E+08	1.55E+06	1.36E+08
cs141	1.19E+06	1.04E+08	1.23E+06	1.08E+08
eu151	0.00E+00	0.00E+00	0.00E+00	0.00E+00
eu153	0.00E+00	0.00E+00	0.00E+00	0.00E+00
eu154	1.05E+04	9.18E+05	8.41E+03	7.39E+05
eu155	5.55E+03	4.85E+05	4.64E+03	4.08E+05
gd154	0.00E+00	0.00E+00	0.00E+00	0.00E+00
gd155	0.00E+00	0.00E+00	0.00E+00	0.00E+00
gd156	0.00E+00	0.00E+00	0.00E+00	0.00E+00
gd157	0.00E+00	0.00E+00	0.00E+00	0.00E+00
gd158	0.00E+00	0.00E+00	0.00E+00	0.00E+00

Table 10 (continued)

Nuclide	MOX core		LEU core	
	Curies/MTHM	Total curies	Curies/MTHM	Total curies
i129	3.85E-02	3.36E+00	3.19E-02	2.80E+00
i130	2.30E+04	2.00E+06	2.04E+04	1.79E+06
i131	1.08E+06	9.43E+07	1.05E+06	9.26E+07
i132	1.57E+06	1.37E+08	1.54E+06	1.35E+08
i133	2.14E+06	1.87E+08	2.15E+06	1.89E+08
i134	2.37E+06	2.07E+08	2.40E+06	2.11E+08
i135	2.05E+06	1.79E+08	2.05E+06	1.80E+08
i136m	4.96E+05	4.33E+07	4.77E+05	4.19E+07
kr83m	1.09E+05	9.54E+06	1.24E+05	1.09E+07
kr85	1.25E+04	1.09E+06	1.46E+04	1.29E+06
kr85m	2.25E+05	1.97E+07	2.69E+05	2.36E+07
kr87	4.30E+05	3.75E+07	5.15E+05	4.53E+07
kr88	5.63E+05	4.91E+07	6.82E+05	5.99E+07
kr89	6.90E+05	6.02E+07	8.48E+05	7.46E+07
kr90	6.95E+05	6.07E+07	8.77E+05	7.71E+07
la139	0.00E+00	0.00E+00	0.00E+00	0.00E+00
la140	1.92E+06	1.68E+08	1.98E+06	1.74E+08
la141	1.67E+06	1.45E+08	1.72E+06	1.51E+08
la142	1.59E+06	1.38E+08	1.65E+06	1.45E+08
la143	1.49E+06	1.30E+08	1.58E+06	1.39E+08
mo100	5.31E-13	4.63E-11	5.11E-13	4.49E-11
mo92	0.00E+00	0.00E+00	0.00E+00	0.00E+00
mo94	0.00E+00	0.00E+00	0.00E+00	0.00E+00
mo95	0.00E+00	0.00E+00	0.00E+00	0.00E+00
mo96	0.00E+00	0.00E+00	0.00E+00	0.00E+00
mo97	0.00E+00	0.00E+00	0.00E+00	0.00E+00
mo98	0.00E+00	0.00E+00	0.00E+00	0.00E+00
mo99	1.94E+06	1.69E+08	1.95E+06	1.71E+08
nb95	1.62E+06	1.41E+08	1.74E+06	1.53E+08
nb97	1.73E+06	1.51E+08	1.79E+06	1.57E+08
nb97m	2.73E+03	2.38E+05	2.36E+03	2.07E+05
nd142	0.00E+00	0.00E+00	0.00E+00	0.00E+00
nd143	0.00E+00	0.00E+00	0.00E+00	0.00E+00
nd144	1.19E-09	1.04E-07	1.23E-09	1.08E-07
nd145	0.00E+00	0.00E+00	0.00E+00	0.00E+00
nd146	0.00E+00	0.00E+00	0.00E+00	0.00E+00
nd147	6.69E+05	5.84E+07	6.81E+05	5.99E+07
nd148	0.00E+00	0.00E+00	0.00E+00	0.00E+00
nd150	7.03E-14	6.14E-12	6.31E-14	5.55E-12
np237	3.20E-01	2.79E+01	3.94E-01	3.47E+01
np238	3.72E+05	3.24E+07	4.51E+05	3.97E+07
np239	2.18E+07	1.91E+09	2.17E+07	1.90E+09
np240	1.50E+04	1.31E+06	1.49E+04	1.31E+06
pd105	0.00E+00	0.00E+00	0.00E+00	0.00E+00
pd107	1.89E-01	1.65E+01	1.25E-01	1.10E+01
pd108	0.00E+00	0.00E+00	0.00E+00	0.00E+00
pd110	0.00E+00	0.00E+00	0.00E+00	0.00E+00
pm147	1.90E+05	1.66E+07	1.98E+05	1.74E+07
pr143	1.45E+06	1.27E+08	1.54E+06	1.35E+08

Table 10 (continued)

Nuclide	MOX core		LEU core	
	Curies/MTHM	Total curies	Curies/MTHM	Total curies
pr144	1.26E+06	1.10E+08	1.36E+06	1.20E+08
pr145	1.04E+06	9.05E+07	1.09E+06	9.56E+07
pu237	8.37E+00	7.30E+02	7.38E+00	6.49E+02
pu238	3.54E+03	3.09E+05	3.57E+03	3.14E+05
pu239	6.30E+02	5.50E+04	3.93E+02	3.46E+04
pu240	1.02E+03	8.95E+04	5.31E+02	4.67E+04
pu241	2.63E+05	2.30E+07	1.54E+05	1.35E+07
pu242	3.22E+00	2.81E+02	2.09E+00	1.84E+02
pu243	6.16E+05	5.38E+07	4.27E+05	3.75E+07
pu244	1.25E-06	1.09E-04	7.35E-07	6.46E-05
pu245	6.00E+00	5.24E+02	3.43E+00	3.01E+02
rb86	2.68E+03	2.34E+05	3.00E+03	2.64E+05
rb88	5.76E+05	5.03E+07	6.95E+05	6.11E+07
rb89	7.53E+05	6.58E+07	9.13E+05	8.03E+07
rb90	7.37E+05	6.43E+07	9.20E+05	8.09E+07
rb90m	1.67E+05	1.46E+07	1.79E+05	1.57E+07
rb91	9.37E+05	8.18E+07	1.12E+06	9.83E+07
rh103m	1.86E+06	1.62E+08	1.67E+06	1.47E+08
rh105	1.33E+06	1.16E+08	1.11E+06	9.77E+07
rh105m	4.02E+05	3.51E+07	3.39E+05	2.98E+07
rh106	8.99E+05	7.85E+07	6.76E+05	5.94E+07
rh107	8.41E+05	7.35E+07	6.73E+05	5.92E+07
ru101	0.00E+00	0.00E+00	0.00E+00	0.00E+00
ru102	0.00E+00	0.00E+00	0.00E+00	0.00E+00
ru103	1.88E+06	1.64E+08	1.69E+06	1.49E+08
ru104	0.00E+00	0.00E+00	0.00E+00	0.00E+00
ru105	1.43E+06	1.24E+08	1.20E+06	1.06E+08
ru106	8.11E+05	7.08E+07	5.94E+05	5.22E+07
ru107	8.29E+05	7.24E+07	6.63E+05	5.83E+07
sb125	1.30E+04	1.13E+06	1.06E+04	9.36E+05
sb127	1.15E+05	1.00E+07	1.02E+05	9.00E+06
sb129	3.38E+05	2.95E+07	3.09E+05	2.72E+07
sb130	3.10E+05	2.70E+07	3.01E+05	2.65E+07
sb130m	3.16E+05	2.76E+07	3.15E+05	2.77E+07
sb133	5.98E+05	5.22E+07	6.38E+05	5.61E+07
se84	1.74E+05	1.52E+07	2.03E+05	1.78E+07
sm147	1.82E-06	1.59E-04	1.85E-06	1.62E-04
sm148	6.08E-11	5.30E-09	5.65E-11	4.97E-09
sm149	0.00E+00	0.00E+00	0.00E+00	0.00E+00
sm150	0.00E+00	0.00E+00	0.00E+00	0.00E+00
sm151	4.81E+02	4.20E+04	3.86E+02	3.39E+04
sm152	0.00E+00	0.00E+00	0.00E+00	0.00E+00
sm154	0.00E+00	0.00E+00	0.00E+00	0.00E+00
sn130	1.85E+05	1.61E+07	1.90E+05	1.67E+07
sr89	7.79E+05	6.80E+07	9.51E+05	8.36E+07
sr90	7.66E+04	6.69E+06	9.03E+04	7.93E+06
sr91	1.02E+06	8.93E+07	1.20E+06	1.06E+08
sr92	1.14E+06	9.98E+07	1.31E+06	1.15E+08
sr93	1.32E+06	1.15E+08	1.46E+06	1.29E+08



Table 10 (continued)

Nuclide	MOX core		LEU core	
	Curies/MTHM	Total curies	Curies/MTHM	Total curies
sr94	1.31E+06	1.14E+08	1.45E+06	1.27E+08
tc101	1.85E+06	1.61E+08	1.82E+06	1.60E+08
tc99	1.62E+01	1.42E+03	1.60E+01	1.40E+03
tc99m	1.71E+06	1.49E+08	1.73E+06	1.52E+08
tel25m	2.84E+03	2.48E+05	2.30E+03	2.02E+05
tel27	1.12E+05	9.77E+06	9.98E+04	8.77E+06
tel27m	3.05E+03	2.66E+05	2.48E+03	2.18E+05
tel29	3.39E+05	2.96E+07	3.09E+05	2.72E+07
tel29m	1.65E+02	1.44E+04	1.38E+02	1.22E+04
tel31	9.53E+05	8.32E+07	9.42E+05	8.28E+07
tel31m	1.70E+05	1.49E+07	1.52E+05	1.33E+07
tel32	1.52E+06	1.33E+08	1.50E+06	1.32E+08
tel33	1.21E+06	1.05E+08	1.23E+06	1.08E+08
tel33m	8.85E+05	7.73E+07	8.98E+05	7.90E+07
tel34	1.79E+06	1.56E+08	1.88E+06	1.66E+08
u234	8.46E-01	7.38E+01	1.40E+00	1.23E+02
u235	1.72E-02	1.50E+00	2.95E-02	2.59E+00
u236	2.37E-01	2.07E+01	3.52E-01	3.09E+01
u237	7.42E+05	6.48E+07	9.82E+05	8.63E+07
u238	3.12E-01	2.72E+01	3.12E-01	2.74E+01
u239	2.19E+07	1.91E+09	2.17E+07	1.91E+09
xel131m	1.43E+04	1.25E+06	1.38E+04	1.22E+06
xel133	2.06E+06	1.80E+08	2.06E+06	1.82E+08
xel133m	2.84E+04	2.48E+06	2.74E+04	2.41E+06
xel135	6.90E+05	6.03E+07	6.02E+05	5.30E+07
xel135m	3.49E+05	3.05E+07	3.32E+05	2.92E+07
xel137	1.91E+06	1.67E+08	1.93E+06	1.69E+08
xel138	1.76E+06	1.53E+08	1.81E+06	1.59E+08
xel139	1.24E+06	1.09E+08	1.32E+06	1.16E+08
xel140	8.08E+05	7.06E+07	8.88E+05	7.81E+07
y90	7.93E+04	6.92E+06	9.34E+04	8.21E+06
y91	1.05E+06	9.17E+07	1.24E+06	1.09E+08
y91m	5.86E+05	5.12E+07	6.89E+05	6.06E+07
y92	1.16E+06	1.01E+08	1.32E+06	1.16E+08
y93	1.35E+06	1.18E+08	1.50E+06	1.32E+08
y94	1.46E+06	1.27E+08	1.59E+06	1.40E+08
y95	1.56E+06	1.36E+08	1.67E+06	1.47E+08
y96	9.08E+05	7.93E+07	9.96E+05	8.75E+07
zr95	1.61E+06	1.41E+08	1.73E+06	1.52E+08
zr97	1.72E+06	1.50E+08	1.77E+06	1.56E+08

Figure 15 graphically presents the MOX/LEU curie ratios of the total core data presented in Table 10.

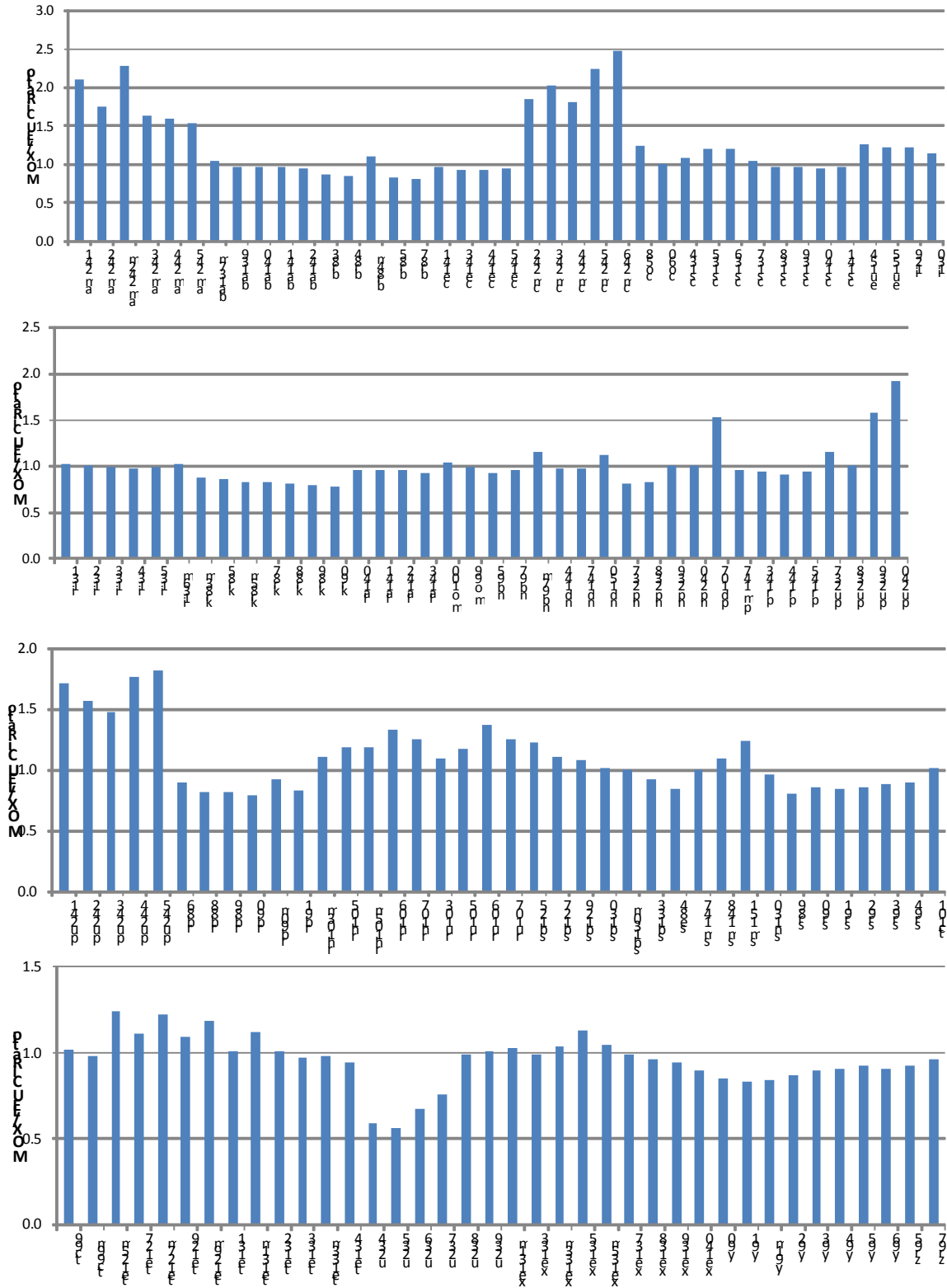


Figure 15. SQN MOX/LEU core-average curie ratios.

**BFN**

Table 11 presents a comparison of the contribution of each nuclide to the radioactivity source in the BFN equilibrium MOX and LEU cores. The activity in curies/MTHM as output by SCALE/ORIGEN has been multiplied by the number of metric tonnes of heavy metal in each core to determine the core total activity. There are 138.67 MTHM in the LEU core and 137.00 MTHM in the MOX core.

**Table 11. Comparison of BFN radioactivity sources**

Nuclide	MOX core		LEU core	
	Curies/MTHM	Total curies	Curies/MTHM	Total curies
am241	2.54E+02	3.48E+04	1.39E+02	1.92E+04
am242	1.08E+05	1.48E+07	7.01E+04	9.72E+06
am242m	1.03E+01	1.41E+03	5.46E+00	7.57E+02
am243	2.58E+01	3.53E+03	2.03E+01	2.81E+03
am244	7.03E+03	9.63E+05	5.82E+03	8.06E+05
am245	1.04E+01	1.43E+03	9.68E+00	1.34E+03
ba137m	1.03E+05	1.41E+07	1.11E+05	1.53E+07
ba139	1.39E+06	1.91E+08	1.42E+06	1.97E+08
ba140	1.34E+06	1.84E+08	1.37E+06	1.90E+08
ba141	1.25E+06	1.72E+08	1.28E+06	1.77E+08
ba142	1.17E+06	1.60E+08	1.20E+06	1.67E+08
br83	8.86E+04	1.21E+07	9.60E+04	1.33E+07
br84	1.48E+05	2.03E+07	1.63E+05	2.26E+07
br84m	7.08E+03	9.71E+05	6.57E+03	9.11E+05
br85	1.96E+05	2.68E+07	2.19E+05	3.04E+07
br87	2.84E+05	3.89E+07	3.21E+05	4.45E+07
cd112	0.00E+00	0.00E+00	0.00E+00	0.00E+00
cd114	0.00E+00	0.00E+00	0.00E+00	0.00E+00
ce140	0.00E+00	0.00E+00	0.00E+00	0.00E+00
ce141	1.27E+06	1.73E+08	1.29E+06	1.79E+08
ce142	0.00E+00	0.00E+00	0.00E+00	0.00E+00
ce143	1.16E+06	1.59E+08	1.21E+06	1.67E+08
ce144	9.84E+05	1.35E+08	1.06E+06	1.47E+08
ce145	7.94E+05	1.09E+08	8.20E+05	1.14E+08
cm242	6.59E+04	9.02E+06	4.27E+04	5.92E+06
cm243	2.39E+01	3.28E+03	1.55E+01	2.14E+03
cm244	3.70E+03	5.07E+05	3.06E+03	4.24E+05
cm245	4.58E-01	6.28E+01	3.35E-01	4.65E+01
cm246	1.03E-01	1.41E+01	8.71E-02	1.21E+01
co58	2.21E-13	3.03E-11	1.83E-13	2.53E-11
co60	1.11E-11	1.52E-09	1.23E-11	1.71E-09
cs133	0.00E+00	0.00E+00	0.00E+00	0.00E+00
cs134	1.42E+05	1.94E+07	1.60E+05	2.22E+07
cs135	6.09E-01	8.34E+01	5.91E-01	8.19E+01
cs136	3.85E+04	5.27E+06	3.80E+04	5.27E+06
cs137	1.08E+05	1.48E+07	1.16E+05	1.62E+07
cs138	1.46E+06	2.01E+08	1.49E+06	2.06E+08
cs139	1.36E+06	1.87E+08	1.39E+06	1.92E+08
cs140	1.13E+06	1.55E+08	1.17E+06	1.63E+08
cs141	9.02E+05	1.24E+08	9.22E+05	1.28E+08
eu151	0.00E+00	0.00E+00	0.00E+00	0.00E+00

Table 11 (continued)

Nuclide	MOX core		LEU core	
	Curies/MTHM	Total curies	Curies/MTHM	Total curies
eu153	0.00E+00	0.00E+00	0.00E+00	0.00E+00
eu154	6.80E+03	9.32E+05	6.44E+03	8.93E+05
eu155	4.26E+03	5.84E+05	4.12E+03	5.71E+05
gd154	0.00E+00	0.00E+00	0.00E+00	0.00E+00
gd155	0.00E+00	0.00E+00	0.00E+00	0.00E+00
gd156	0.00E+00	0.00E+00	0.00E+00	0.00E+00
gd157	0.00E+00	0.00E+00	0.00E+00	0.00E+00
gd158	0.00E+00	0.00E+00	0.00E+00	0.00E+00
il29	3.11E-02	4.26E+00	2.90E-02	4.02E+00
il30	1.59E+04	2.18E+06	1.63E+04	2.26E+06
il31	7.93E+05	1.09E+08	7.78E+05	1.08E+08
il32	1.15E+06	1.58E+08	1.14E+06	1.58E+08
il33	1.59E+06	2.18E+08	1.60E+06	2.22E+08
il34	1.78E+06	2.43E+08	1.79E+06	2.49E+08
il35	1.52E+06	2.09E+08	1.52E+06	2.11E+08
il36m	3.59E+05	4.92E+07	3.49E+05	4.83E+07
kr83m	8.79E+04	1.20E+07	9.54E+04	1.32E+07
kr85	1.09E+04	1.49E+06	1.36E+04	1.89E+06
kr85m	1.89E+05	2.59E+07	2.12E+05	2.93E+07
kr87	3.62E+05	4.96E+07	4.07E+05	5.64E+07
kr88	4.78E+05	6.54E+07	5.40E+05	7.48E+07
kr89	5.92E+05	8.11E+07	6.75E+05	9.36E+07
kr90	6.07E+05	8.32E+07	7.02E+05	9.74E+07
la139	0.00E+00	0.00E+00	0.00E+00	0.00E+00
la140	1.40E+06	1.91E+08	1.43E+06	1.98E+08
la141	1.26E+06	1.73E+08	1.29E+06	1.78E+08
la142	1.21E+06	1.66E+08	1.24E+06	1.72E+08
la143	1.15E+06	1.58E+08	1.20E+06	1.66E+08
mo100	4.43E-13	6.06E-11	4.78E-13	6.63E-11
mo92	0.00E+00	0.00E+00	0.00E+00	0.00E+00
mo94	0.00E+00	0.00E+00	0.00E+00	0.00E+00
mo95	0.00E+00	0.00E+00	0.00E+00	0.00E+00
mo96	0.00E+00	0.00E+00	0.00E+00	0.00E+00
mo97	0.00E+00	0.00E+00	0.00E+00	0.00E+00
mo98	0.00E+00	0.00E+00	0.00E+00	0.00E+00
mo99	1.44E+06	1.97E+08	1.45E+06	2.01E+08
nb95	1.26E+06	1.73E+08	1.32E+06	1.84E+08
nb97	1.31E+06	1.80E+08	1.34E+06	1.86E+08
nb97m	1.84E+03	2.52E+05	1.63E+03	2.26E+05
nd142	0.00E+00	0.00E+00	0.00E+00	0.00E+00
nd143	0.00E+00	0.00E+00	0.00E+00	0.00E+00
nd144	1.03E-09	1.42E-07	1.28E-09	1.77E-07
nd145	0.00E+00	0.00E+00	0.00E+00	0.00E+00
nd146	0.00E+00	0.00E+00	0.00E+00	0.00E+00
nd147	5.05E+05	6.91E+07	5.12E+05	7.09E+07
nd148	0.00E+00	0.00E+00	0.00E+00	0.00E+00
nd150	5.76E-14	7.89E-12	5.85E-14	8.11E-12
np237	2.30E-01	3.15E+01	3.15E-01	4.37E+01
np238	1.92E+05	2.63E+07	2.74E+05	3.80E+07

Table 11 (continued)

Nuclide	MOX core		LEU core	
	Curies/MTHM	Total curies	Curies/MTHM	Total curies
np239	1.37E+07	1.88E+09	1.40E+07	1.94E+09
np240	6.66E+03	9.13E+05	7.04E+03	9.76E+05
pd105	0.00E+00	0.00E+00	0.00E+00	0.00E+00
pd107	1.46E-01	2.00E+01	1.09E-01	1.51E+01
pd108	0.00E+00	0.00E+00	0.00E+00	0.00E+00
pd110	0.00E+00	0.00E+00	0.00E+00	0.00E+00
pm147	1.77E+05	2.43E+07	1.87E+05	2.59E+07
pr143	1.14E+06	1.57E+08	1.19E+06	1.65E+08
pr144	9.90E+05	1.36E+08	1.06E+06	1.48E+08
pr145	7.94E+05	1.09E+08	8.21E+05	1.14E+08
pu237	3.62E+00	4.96E+02	4.08E+00	5.65E+02
pu238	2.20E+03	3.02E+05	2.75E+03	3.81E+05
pu239	4.57E+02	6.26E+04	3.03E+02	4.20E+04
pu240	8.67E+02	1.19E+05	4.85E+02	6.73E+04
pu241	1.71E+05	2.35E+07	1.09E+05	1.52E+07
pu242	2.48E+00	3.39E+02	1.84E+00	2.55E+02
pu243	3.12E+05	4.27E+07	2.51E+05	3.49E+07
pu244	5.85E-07	8.02E-05	4.91E-07	6.81E-05
pu245	1.75E+00	2.40E+02	1.53E+00	2.13E+02
rb86	1.55E+03	2.13E+05	1.99E+03	2.75E+05
rb88	4.87E+05	6.67E+07	5.49E+05	7.61E+07
rb89	6.40E+05	8.76E+07	7.23E+05	1.00E+08
rb90	6.39E+05	8.75E+07	7.35E+05	1.02E+08
rb90m	1.30E+05	1.79E+07	1.36E+05	1.89E+07
rb91	7.86E+05	1.08E+08	8.81E+05	1.22E+08
rh103m	1.30E+06	1.78E+08	1.20E+06	1.66E+08
rh105	8.78E+05	1.20E+08	7.64E+05	1.06E+08
rh105m	2.66E+05	3.64E+07	2.33E+05	3.23E+07
rh106	6.14E+05	8.41E+07	4.96E+05	6.88E+07
rh107	5.41E+05	7.42E+07	4.52E+05	6.27E+07
ru101	0.00E+00	0.00E+00	0.00E+00	0.00E+00
ru102	0.00E+00	0.00E+00	0.00E+00	0.00E+00
ru103	1.31E+06	1.80E+08	1.21E+06	1.68E+08
ru104	0.00E+00	0.00E+00	0.00E+00	0.00E+00
ru105	9.44E+05	1.29E+08	8.26E+05	1.15E+08
ru106	5.62E+05	7.71E+07	4.46E+05	6.19E+07
ru107	5.33E+05	7.30E+07	4.46E+05	6.18E+07
sb125	9.92E+03	1.36E+06	8.90E+03	1.23E+06
sb127	7.89E+04	1.08E+07	7.24E+04	1.00E+07
sb129	2.36E+05	3.23E+07	2.20E+05	3.05E+07
sb130	2.25E+05	3.09E+07	2.21E+05	3.06E+07
sb130m	2.34E+05	3.21E+07	2.33E+05	3.23E+07
sb133	4.63E+05	6.34E+07	4.85E+05	6.72E+07
se84	1.43E+05	1.96E+07	1.58E+05	2.19E+07
sm147	2.01E-06	2.75E-04	2.29E-06	3.17E-04
sm148	4.38E-11	6.00E-09	5.02E-11	6.97E-09
sm149	0.00E+00	0.00E+00	0.00E+00	0.00E+00
sm150	0.00E+00	0.00E+00	0.00E+00	0.00E+00
sm151	3.57E+02	4.89E+04	3.11E+02	4.32E+04

Table 11 (continued)

Nuclide	MOX core		LEU core	
	Curies/MTHM	Total curies	Curies/MTHM	Total curies
sm152	0.00E+00	0.00E+00	0.00E+00	0.00E+00
sm154	0.00E+00	0.00E+00	0.00E+00	0.00E+00
sn130	1.40E+05	1.91E+07	1.43E+05	1.98E+07
sr89	6.60E+05	9.04E+07	7.49E+05	1.04E+08
sr90	6.72E+04	9.20E+06	8.58E+04	1.19E+07
sr91	8.50E+05	1.16E+08	9.44E+05	1.31E+08
sr92	9.31E+05	1.27E+08	1.02E+06	1.41E+08
sr93	1.05E+06	1.44E+08	1.13E+06	1.56E+08
sr94	1.04E+06	1.43E+08	1.11E+06	1.54E+08
tc101	1.36E+06	1.86E+08	1.34E+06	1.86E+08
tc99	1.40E+01	1.92E+03	1.52E+01	2.11E+03
tc99m	1.28E+06	1.75E+08	1.28E+06	1.78E+08
te125m	2.19E+03	3.01E+05	1.97E+03	2.73E+05
te127	7.73E+04	1.06E+07	7.07E+04	9.81E+06
te127m	2.19E+03	3.00E+05	1.85E+03	2.56E+05
te129	2.36E+05	3.23E+07	2.20E+05	3.05E+07
te129m	1.05E+02	1.44E+04	9.46E+01	1.31E+04
te131	7.01E+05	9.61E+07	6.95E+05	9.63E+07
te131m	1.17E+05	1.60E+07	1.07E+05	1.48E+07
te132	1.12E+06	1.53E+08	1.11E+06	1.54E+08
te133	9.08E+05	1.24E+08	9.22E+05	1.28E+08
te133m	6.65E+05	9.11E+07	6.71E+05	9.31E+07
te134	1.38E+06	1.89E+08	1.43E+06	1.98E+08
u234	1.09E+00	1.49E+02	1.50E+00	2.08E+02
u235	2.55E-02	3.49E+00	3.14E-02	4.36E+00
u236	2.30E-01	3.15E+01	3.40E-01	4.72E+01
u237	4.37E+05	5.98E+07	5.95E+05	8.25E+07
u238	3.14E-01	4.31E+01	3.13E-01	4.35E+01
u239	1.37E+07	1.88E+09	1.41E+07	1.95E+09
xel131m	1.04E+04	1.42E+06	1.02E+04	1.41E+06
xel133	1.54E+06	2.11E+08	1.54E+06	2.14E+08
xel133m	2.07E+04	2.83E+06	2.03E+04	2.81E+06
xel135	5.14E+05	7.04E+07	4.61E+05	6.39E+07
xel135m	2.50E+05	3.43E+07	2.41E+05	3.34E+07
xel137	1.43E+06	1.96E+08	1.43E+06	1.99E+08
xel138	1.33E+06	1.83E+08	1.36E+06	1.89E+08
xel139	9.64E+05	1.32E+08	1.01E+06	1.39E+08
xel140	6.40E+05	8.76E+07	6.82E+05	9.46E+07
y90	6.92E+04	9.49E+06	8.87E+04	1.23E+07
y91	8.70E+05	1.19E+08	9.71E+05	1.35E+08
y91m	4.87E+05	6.67E+07	5.40E+05	7.49E+07
y92	9.41E+05	1.29E+08	1.03E+06	1.43E+08
y93	1.08E+06	1.48E+08	1.16E+06	1.60E+08
y94	1.15E+06	1.58E+08	1.22E+06	1.69E+08
y95	1.21E+06	1.66E+08	1.27E+06	1.76E+08
y96	7.16E+05	9.81E+07	7.62E+05	1.06E+08
zr95	1.26E+06	1.72E+08	1.32E+06	1.83E+08
zr97	1.30E+06	1.78E+08	1.33E+06	1.84E+08

Figure 16 graphically presents the MOX/LEU curie ratios of the total core data presented in Table 11.

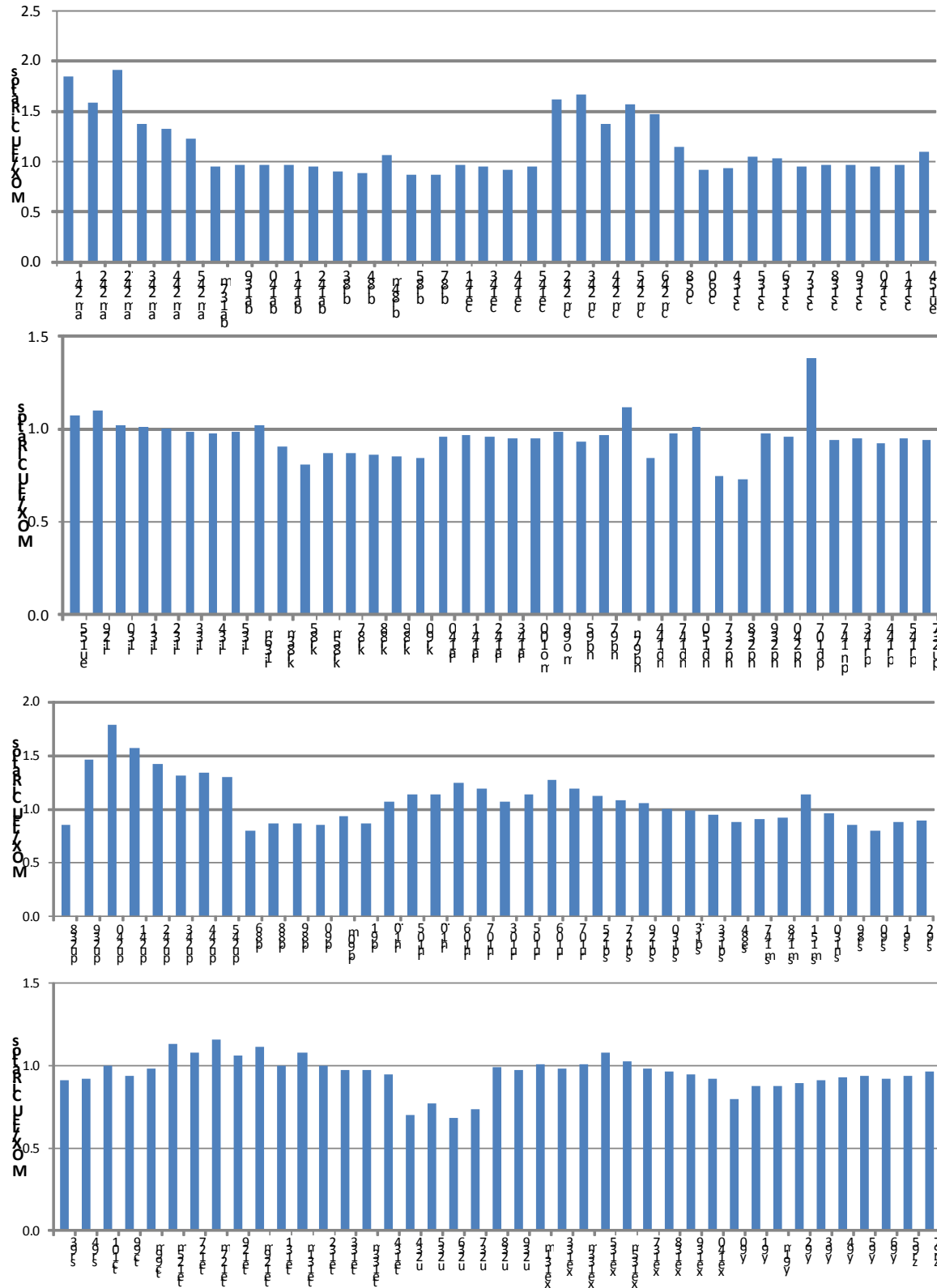


Figure 16. BFN MOX/LEU core-average curie ratios.

## 9.5 Decay Heat

### SQN

The total EOC decay heat trajectories were calculated for each burnup group of each of the SQN MOX and LEU assembly types. This was accomplished by taking the fuel composition at the burnup step closest to the average burnup of each group and simulating a 100-year decay period. The group-average burnups were not exactly the same as those of any particular burnup step, and the next higher burnup value was chosen as the starting point for the decays. Figure 17 presents a comparison of the individual decay heat trajectories, in W/MTHM, of the fuel assemblies in the SQN equilibrium LEU core over a 100-year period (expressed in days) following discharge from the core. Due to the proprietary nature of the fuel rod loadings, the enrichments have been replaced with letter designations (A, B, etc.). The numerical values are summarized in Table 12. Figure 18, Table 13, and Figure 19, Table 14 present similar comparisons of the individual decay heat trajectories of the LEU and MOX fuel assemblies in the SQN equilibrium MOX core over a 100-year period following discharge from the core. (In the tables and figures, an LEU fuel assembly identification is preceded by an “L.” A MOX fuel assembly identification is preceded by an “M.” The three burnup classes, low, medium, and high, are designated b1, b2, and b3, respectively.)

The total EOC decay heat trajectories, in watts, for the SQN MOX and LEU cores were obtained by creating new TRITON ft71001 nuclide concentration files for the MOX and LEU cores using the core weighting fractions shown in Table 1 and Table 2, respectively. The new ft71001 files contain the nuclide concentrations of each assembly type, at the burnup step with the burnup closest to the average burnup of a burnup class weighted by their core fractions. The total core average decay heat as a function of time was then calculated by ORIGEN using the core-average set of nuclide concentrations in the new ft71001 files.

Figure 20 presents the core total decay heat curves decayed for a period of 1 year. The decay heat data are reproduced in tabular form in Table 15.



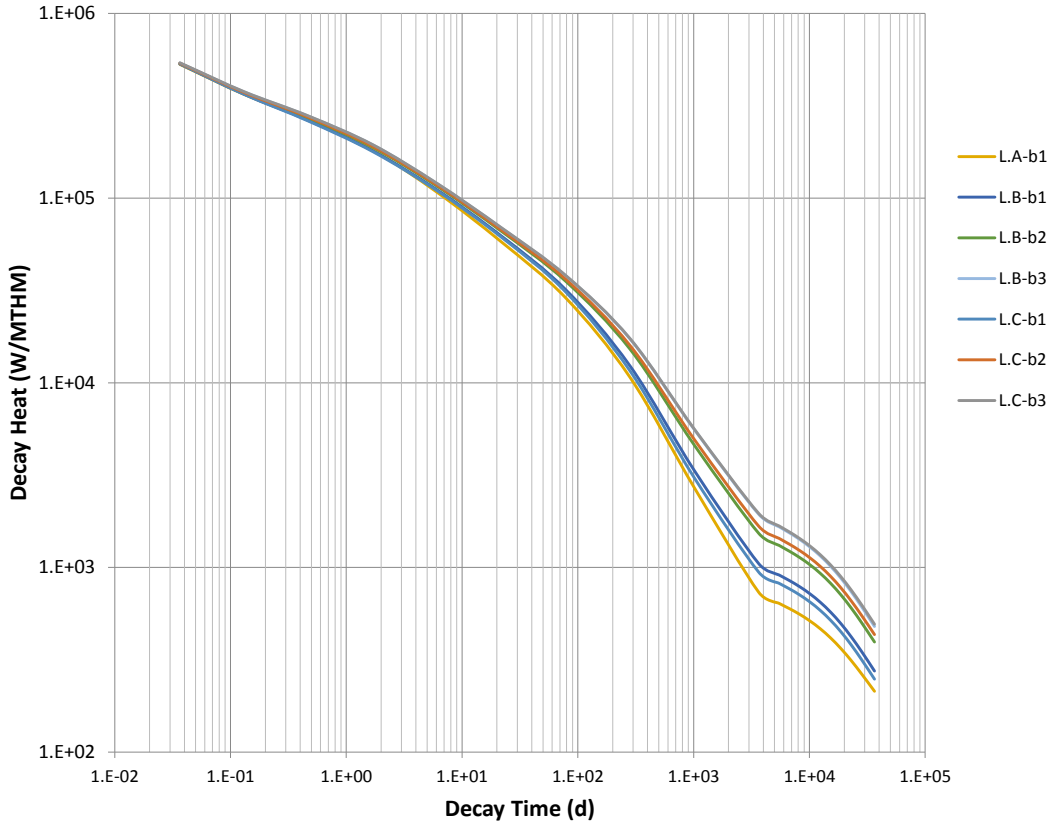


Figure 17. Comparison of assembly decay heat trajectories in the SQN equilibrium LEU core – 100 years.

Table 12. Decay heat (W/MTHM) for the fuel assemblies in the SQN LEU core

Time (d)	LA-b1	LB-b1	LB-b2	LB-b3	LC-b1	LC-b2	LC-b3
0.00E+00	2.36E+06	2.40E+06	2.38E+06	2.37E+06	2.41E+06	2.38E+06	2.37E+06
3.65E-02	5.31E+05	5.33E+05	5.36E+05	5.39E+05	5.34E+05	5.38E+05	5.40E+05
1.31E-01	3.65E+05	3.66E+05	3.71E+05	3.76E+05	3.65E+05	3.74E+05	3.77E+05
4.72E-01	2.66E+05	2.63E+05	2.72E+05	2.78E+05	2.62E+05	2.74E+05	2.78E+05
1.70E+00	1.82E+05	1.80E+05	1.88E+05	1.94E+05	1.79E+05	1.91E+05	1.95E+05
6.09E+00	1.08E+05	1.11E+05	1.16E+05	1.20E+05	1.10E+05	1.18E+05	1.20E+05
2.19E+01	5.80E+04	6.23E+04	6.61E+04	6.89E+04	6.14E+04	6.72E+04	6.91E+04
7.87E+01	2.88E+04	3.19E+04	3.54E+04	3.80E+04	3.10E+04	3.64E+04	3.81E+04
2.83E+02	1.07E+04	1.23E+04	1.52E+04	1.73E+04	1.16E+04	1.59E+04	1.73E+04
1.02E+03	2.70E+03	3.35E+03	4.59E+03	5.58E+03	3.06E+03	4.93E+03	5.60E+03
3.65E+03	7.35E+02	1.05E+03	1.53E+03	1.94E+03	9.44E+02	1.67E+03	1.96E+03
5.48E+03	6.40E+02	9.10E+02	1.32E+03	1.66E+03	8.21E+02	1.44E+03	1.68E+03
6.73E+03	5.99E+02	8.49E+02	1.22E+03	1.54E+03	7.67E+02	1.34E+03	1.56E+03
8.26E+03	5.58E+02	7.87E+02	1.13E+03	1.42E+03	7.10E+02	1.24E+03	1.43E+03
1.01E+04	5.13E+02	7.20E+02	1.03E+03	1.29E+03	6.50E+02	1.13E+03	1.31E+03
1.24E+04	4.66E+02	6.48E+02	9.28E+02	1.16E+03	5.85E+02	1.01E+03	1.17E+03
1.53E+04	4.15E+02	5.73E+02	8.19E+02	1.02E+03	5.18E+02	8.95E+02	1.03E+03
1.88E+04	3.64E+02	4.96E+02	7.09E+02	8.75E+02	4.48E+02	7.75E+02	8.89E+02
2.30E+04	3.13E+02	4.21E+02	6.02E+02	7.39E+02	3.80E+02	6.59E+02	7.53E+02
2.83E+04	2.66E+02	3.51E+02	5.02E+02	6.14E+02	3.17E+02	5.50E+02	6.28E+02
3.47E+04	2.23E+02	2.89E+02	4.14E+02	5.04E+02	2.61E+02	4.55E+02	5.17E+02
3.65E+04	2.14E+02	2.75E+02	3.94E+02	4.80E+02	2.49E+02	4.33E+02	4.93E+02

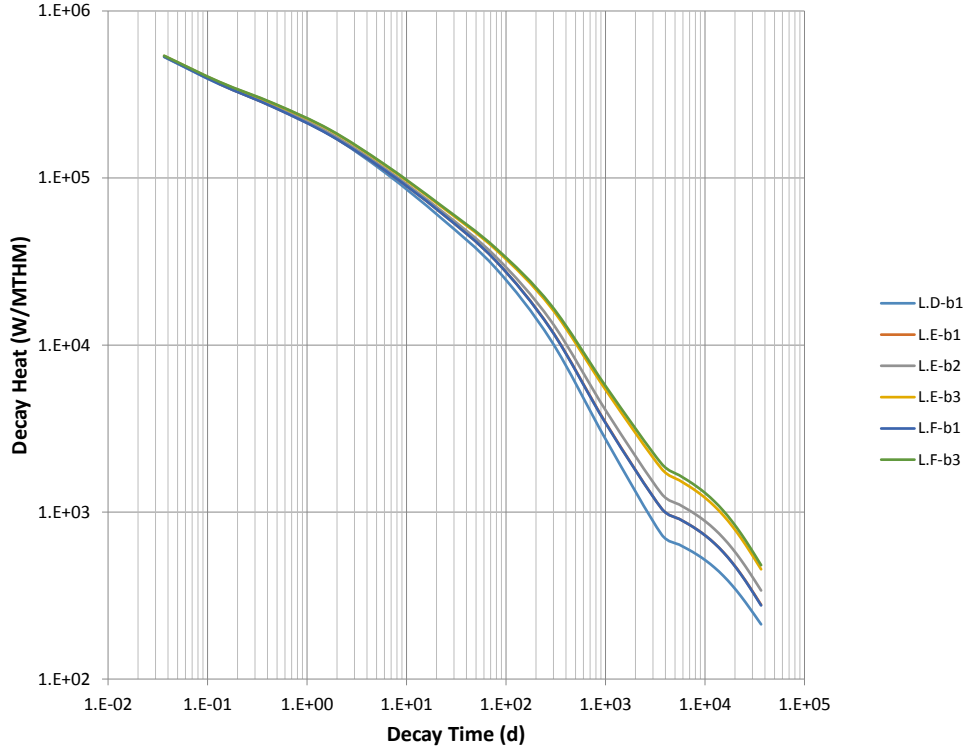


Figure 18. Comparison of assembly decay heat trajectories in the SQN equilibrium MOX core – 100 years, part A.

Table 13. Decay heat (W/MTHM) for the fuel assemblies in the SQN MOX core – part A

Time (d)	LD-b1	LE-b1	LE-b2	LE-b3	LF-b1	LF-b3
0.00E+00	2.36E+06	2.40E+06	2.38E+06	2.37E+06	2.40E+06	2.37E+06
3.65E-02	5.31E+05	5.33E+05	5.35E+05	5.39E+05	5.33E+05	5.40E+05
1.31E-01	3.65E+05	3.66E+05	3.69E+05	3.75E+05	3.66E+05	3.76E+05
4.72E-01	2.65E+05	2.64E+05	2.68E+05	2.77E+05	2.63E+05	2.78E+05
1.70E+00	1.82E+05	1.81E+05	1.85E+05	1.93E+05	1.81E+05	1.95E+05
6.09E+00	1.08E+05	1.11E+05	1.14E+05	1.19E+05	1.11E+05	1.20E+05
2.19E+01	5.80E+04	6.23E+04	6.42E+04	6.80E+04	6.23E+04	6.89E+04
7.87E+01	2.88E+04	3.19E+04	3.38E+04	3.73E+04	3.19E+04	3.80E+04
2.83E+02	1.07E+04	1.24E+04	1.39E+04	1.68E+04	1.24E+04	1.73E+04
1.02E+03	2.69E+03	3.38E+03	4.00E+03	5.32E+03	3.35E+03	5.60E+03
3.65E+03	7.35E+02	1.05E+03	1.29E+03	1.82E+03	1.05E+03	1.95E+03
5.48E+03	6.40E+02	9.12E+02	1.11E+03	1.55E+03	9.11E+02	1.67E+03
6.73E+03	6.00E+02	8.51E+02	1.03E+03	1.44E+03	8.50E+02	1.54E+03
8.26E+03	5.58E+02	7.89E+02	9.57E+02	1.33E+03	7.88E+02	1.42E+03
1.01E+04	5.14E+02	7.22E+02	8.75E+02	1.21E+03	7.21E+02	1.29E+03
1.24E+04	4.66E+02	6.51E+02	7.88E+02	1.08E+03	6.49E+02	1.16E+03
1.53E+04	4.15E+02	5.76E+02	6.97E+02	9.53E+02	5.74E+02	1.02E+03
1.88E+04	3.64E+02	4.99E+02	6.05E+02	8.23E+02	4.97E+02	8.77E+02
2.30E+04	3.13E+02	4.24E+02	5.14E+02	6.96E+02	4.22E+02	7.41E+02
2.83E+04	2.65E+02	3.55E+02	4.30E+02	5.80E+02	3.52E+02	6.16E+02
3.47E+04	2.23E+02	2.93E+02	3.56E+02	4.77E+02	2.90E+02	5.06E+02
3.65E+04	2.13E+02	2.79E+02	3.39E+02	4.54E+02	2.76E+02	4.82E+02

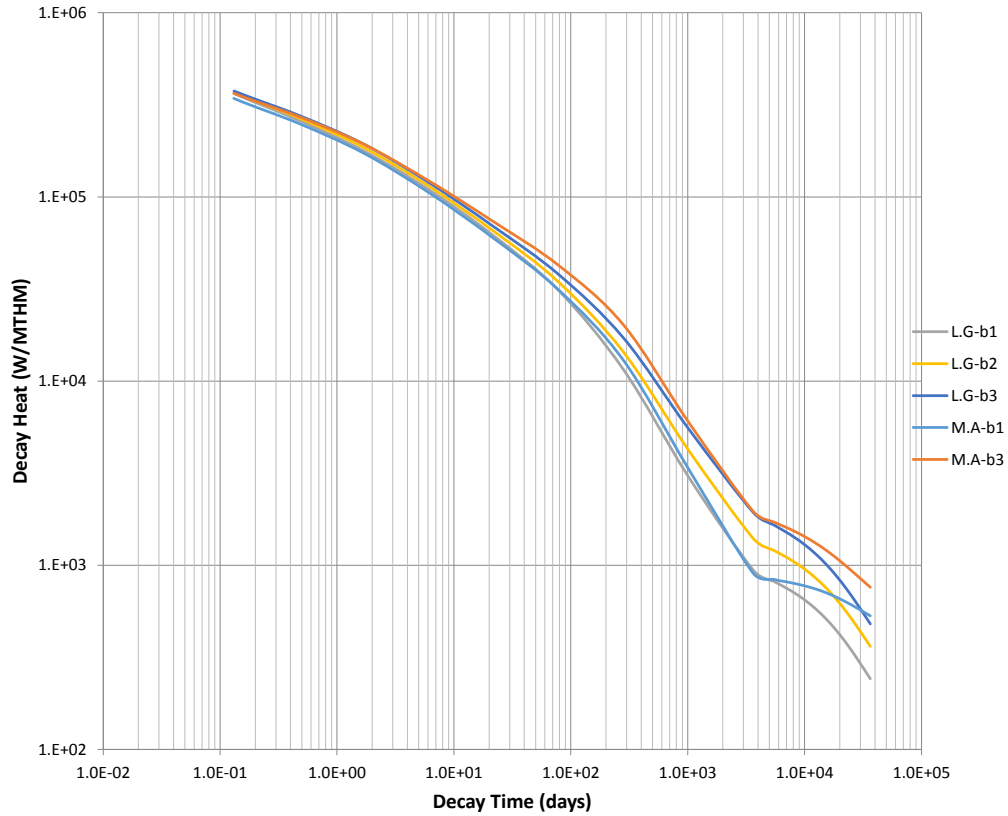


Figure 19. Comparison of assembly decay heat trajectories in the SQN equilibrium MOX core – 100 years, part B.

Table 14. Decay heat (W/MTHM) for the fuel assemblies in the SQN MOX core – part B

Time (d)	LG-b1	LG-b2	LG-b3	MA-b1	MA-b3
0.00E+00	2.41E+06	2.39E+06	2.37E+06	2.22E+06	2.30E+06
3.65E-02	5.33E+05	5.36E+05	5.39E+05	4.98E+05	5.21E+05
1.31E-01	3.64E+05	3.70E+05	3.76E+05	3.43E+05	3.65E+05
4.72E-01	2.60E+05	2.68E+05	2.77E+05	2.51E+05	2.73E+05
1.70E+00	1.78E+05	1.86E+05	1.93E+05	1.73E+05	1.93E+05
6.09E+00	1.10E+05	1.15E+05	1.20E+05	1.06E+05	1.22E+05
2.19E+01	6.15E+04	6.52E+04	6.89E+04	5.96E+04	7.33E+04
7.87E+01	3.10E+04	3.45E+04	3.79E+04	3.13E+04	4.25E+04
2.83E+02	1.15E+04	1.44E+04	1.71E+04	1.30E+04	2.02E+04
1.02E+03	3.02E+03	4.22E+03	5.51E+03	3.35E+03	6.00E+03
3.65E+03	9.38E+02	1.40E+03	1.93E+03	9.03E+02	1.96E+03
5.48E+03	8.16E+02	1.21E+03	1.65E+03	8.38E+02	1.72E+03
6.73E+03	7.62E+02	1.12E+03	1.53E+03	8.18E+02	1.62E+03
8.26E+03	7.05E+02	1.04E+03	1.41E+03	7.97E+02	1.53E+03
1.01E+04	6.44E+02	9.49E+02	1.29E+03	7.73E+02	1.43E+03
1.24E+04	5.80E+02	8.54E+02	1.15E+03	7.44E+02	1.32E+03
1.53E+04	5.11E+02	7.54E+02	1.01E+03	7.10E+02	1.21E+03
1.88E+04	4.42E+02	6.53E+02	8.74E+02	6.71E+02	1.10E+03
2.30E+04	3.74E+02	5.55E+02	7.39E+02	6.29E+02	9.83E+02
2.83E+04	3.10E+02	4.63E+02	6.15E+02	5.85E+02	8.78E+02
3.47E+04	2.54E+02	3.82E+02	5.05E+02	5.42E+02	7.83E+02
3.65E+04	2.42E+02	3.63E+02	4.81E+02	5.32E+02	7.61E+02

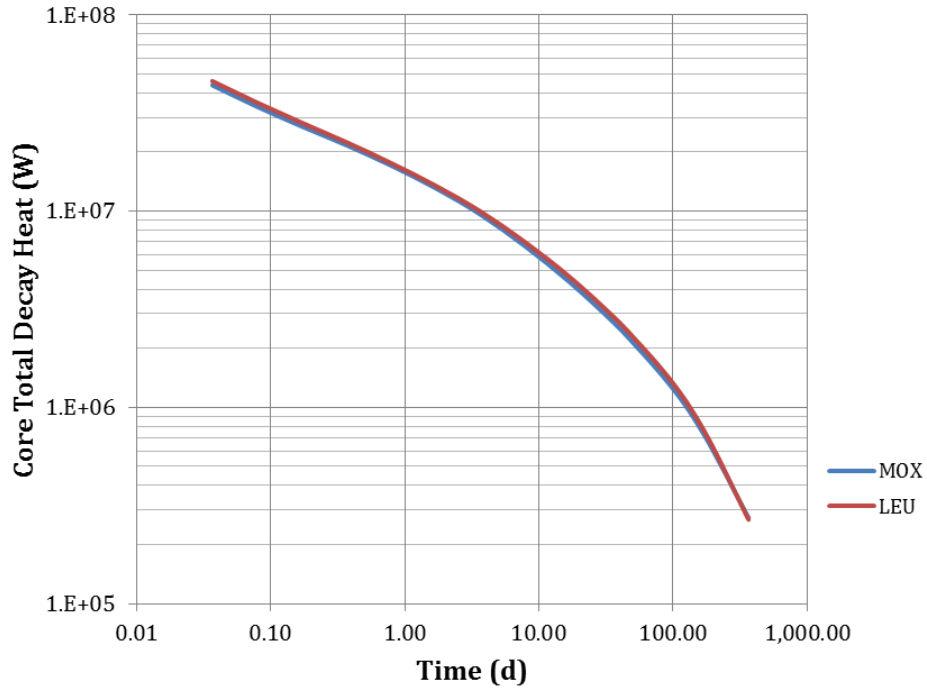


Figure 20. Comparison of decay heat in SQN equilibrium MOX and LEU cores – 1 year.

Table 15. Decay heat for the SQN LEU and MOX cores – 1 year

Time (d)	LEU (W)	MOX (W)	Time (d)	LEU (W)	MOX (W)
0.00E+00	2.19E+08	2.05E+08	5.48E+01	2.14E+06	2.00E+06
3.65E-03	4.60E+07	4.38E+07	6.73E+01	1.83E+06	1.72E+06
1.02E-02	3.58E+07	3.41E+07	8.26E+01	1.56E+06	1.46E+06
2.83E-02	2.82E+07	2.71E+07	1.01E+02	1.31E+06	1.23E+06
7.87E-02	2.25E+07	2.17E+07	1.24E+02	1.08E+06	1.02E+06
2.19E-01	1.76E+07	1.71E+07	1.53E+02	8.64E+05	8.21E+05
6.09E-01	1.35E+07	1.31E+07	1.88E+02	6.73E+05	6.47E+05
1.70E+00	9.97E+06	9.63E+06	2.30E+02	5.12E+05	5.00E+05
4.72E+00	6.95E+06	6.63E+06	2.83E+02	3.84E+05	3.82E+05
1.31E+01	4.61E+06	4.35E+06	3.47E+02	2.88E+05	2.92E+05
3.65E+01	2.86E+06	2.68E+06	3.65E+02	2.68E+05	2.73E+05

## **BFN**

The total EOC decay heat trajectories were calculated for each burnup group of each of the BFN MOX and LEU assembly types. This was accomplished by taking the fuel composition at the burnup step closest to the average burnup of each group and simulating a 100-year decay period. The group-average burnups were not exactly the same as those of a particular burnup step, and the next higher burnup value was chosen as the starting point for the decays. Figure 21 presents a comparison of the individual decay heat trajectories, in W/MTHM, of the “A” fuel assemblies in the BFN equilibrium LEU core over a 100-year period (expressed in days) following discharge from the core. The numerical values are summarized in Table 16. These data are presented for BFN LEU core assemblies “B” and “C” in Figure 22, Table 17 and Figure 23, Table 18, respectively.

Similar data for the BFN equilibrium MOX core LEU fuel assemblies “A” and “B” are presented in Figure 24, Table 19 and Figure 25, Table 20; for MOX fuel assemblies “C” and “D,” data are presented in Figure 26, Table 21 and Figure 27, Table 22, respectively.

The total EOC decay heat trajectories, in watts, for the BFN MOX and LEU cores were obtained by creating new TRITON ft71001 nuclide concentration files for the MOX and LEU cores using the core fractions shown in Table 3 and Table 6, respectively. The new ft71001 files contain the nuclide concentrations of each assembly type, at the burnup step with the burnup closest to the average burnup of a burnup class weighted by their core fractions. The total core average decay heat as a function of time was then calculated by ORIGEN using the core-average set of nuclide concentrations in the new ft71001 files.

Figure 28 presents the core total decay heat curves decayed for a period of 1 year. Since the burnup steps did not, in general, occur at the average burnup of the burnup classes, the next step of burnup greater than the class average was chosen for inclusion in the ft71001 files. The decay heat data are reproduced in tabular form in Table 23.

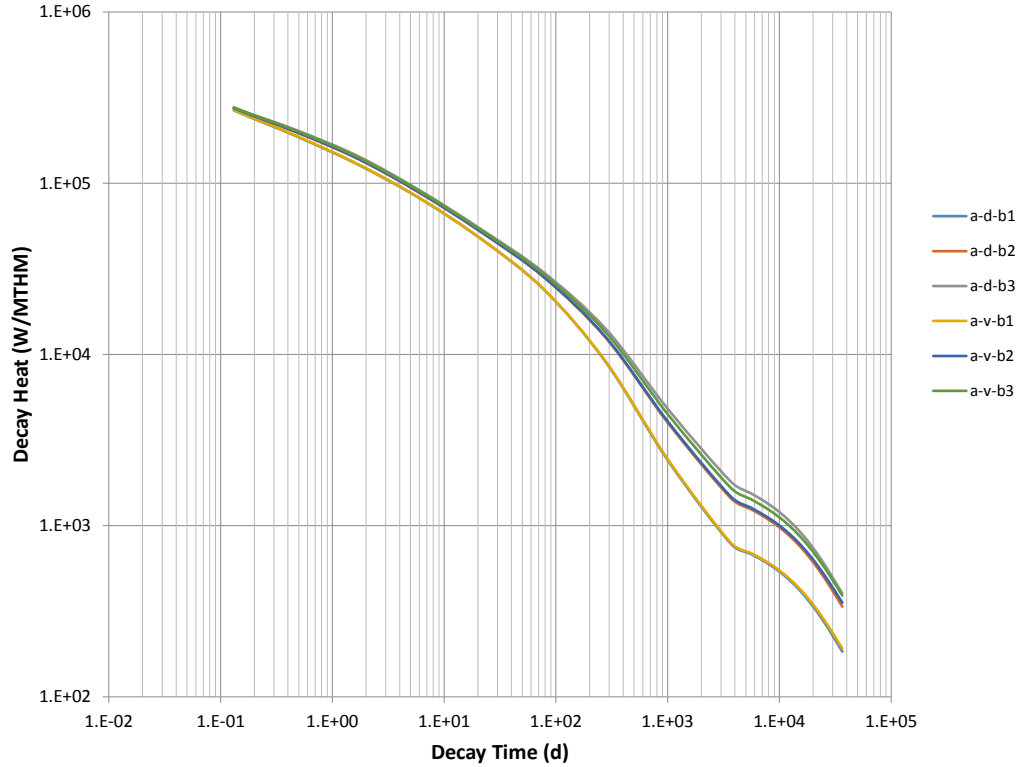


Figure 21. Comparison of “A” assembly decay heat trajectories in the BFN equilibrium LEU core – 100 years.

Table 16. Decay heat (W/MTHM) for the fuel “A” assembly in the BFN LEU core

Time (d)	Dominant zone			Vanished zone		
	b1	b2	b3	b1	b2	b3
0.00E+00	1.80E+06	1.76E+06	1.75E+06	1.81E+06	1.76E+06	1.76E+06
3.65E-02	3.93E+05	3.94E+05	3.97E+05	3.94E+05	3.96E+05	3.97E+05
1.31E-01	2.67E+05	2.73E+05	2.77E+05	2.68E+05	2.74E+05	2.76E+05
4.72E-01	1.89E+05	1.99E+05	2.05E+05	1.90E+05	2.00E+05	2.03E+05
1.70E+00	1.29E+05	1.39E+05	1.44E+05	1.29E+05	1.39E+05	1.42E+05
6.09E+00	8.14E+04	8.75E+04	9.06E+04	8.16E+04	8.79E+04	8.95E+04
2.19E+01	4.68E+04	5.13E+04	5.34E+04	4.69E+04	5.15E+04	5.26E+04
7.87E+01	2.38E+04	2.81E+04	3.00E+04	2.39E+04	2.83E+04	2.93E+04
2.83E+02	8.94E+03	1.24E+04	1.40E+04	9.00E+03	1.25E+04	1.33E+04
1.02E+03	2.38E+03	3.95E+03	4.76E+03	2.41E+03	4.02E+03	4.42E+03
3.65E+03	7.88E+02	1.45E+03	1.81E+03	7.96E+02	1.49E+03	1.67E+03
5.48E+03	6.87E+02	1.25E+03	1.55E+03	6.94E+02	1.28E+03	1.43E+03
6.73E+03	6.39E+02	1.16E+03	1.44E+03	6.47E+02	1.19E+03	1.33E+03
8.26E+03	5.89E+02	1.07E+03	1.32E+03	5.97E+02	1.10E+03	1.22E+03
1.01E+04	5.36E+02	9.71E+02	1.19E+03	5.43E+02	9.96E+02	1.11E+03
1.24E+04	4.78E+02	8.66E+02	1.06E+03	4.86E+02	8.90E+02	9.89E+02
1.53E+04	4.18E+02	7.57E+02	9.21E+02	4.26E+02	7.80E+02	8.65E+02
1.88E+04	3.57E+02	6.47E+02	7.83E+02	3.65E+02	6.69E+02	7.41E+02
2.30E+04	2.98E+02	5.41E+02	6.51E+02	3.05E+02	5.61E+02	6.20E+02
2.83E+04	2.43E+02	4.42E+02	5.30E+02	2.50E+02	4.62E+02	5.09E+02
3.47E+04	1.95E+02	3.56E+02	4.24E+02	2.02E+02	3.75E+02	4.12E+02
3.65E+04	1.84E+02	3.37E+02	4.01E+02	1.91E+02	3.55E+02	3.91E+02

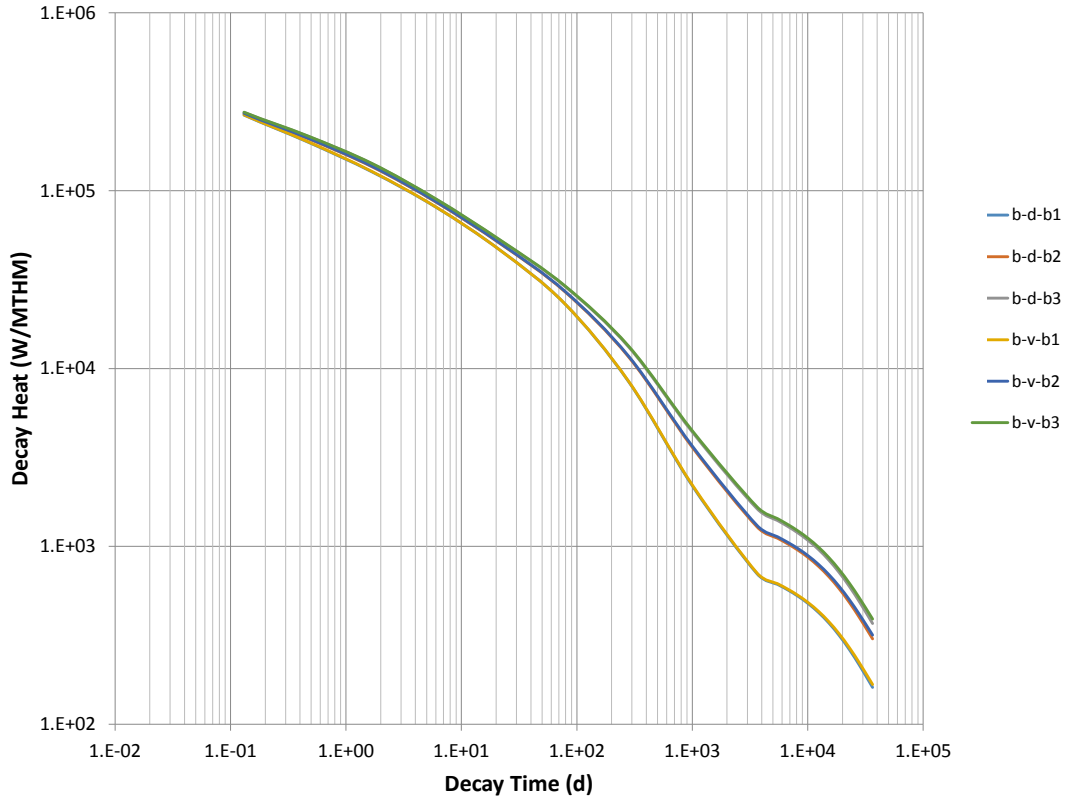


Figure 22. Comparison of “B” assembly decay heat trajectories in the BFN equilibrium LEU core – 100 years.

Table 17. Decay heat (W/MTHM) for the fuel “B” assembly in the BFN LEU core

Time (d)	Dominant zone			Vanished zone		
	b1	b2	b3	b1	b2	b3
0.00E+00	1.81E+06	1.76E+06	1.75E+06	1.81E+06	1.77E+06	1.76E+06
3.65E-02	3.93E+05	3.93E+05	3.96E+05	3.94E+05	3.95E+05	3.97E+05
1.31E-01	2.66E+05	2.71E+05	2.75E+05	2.67E+05	2.72E+05	2.76E+05
4.72E-01	1.87E+05	1.96E+05	2.02E+05	1.88E+05	1.97E+05	2.03E+05
1.70E+00	1.27E+05	1.36E+05	1.41E+05	1.28E+05	1.37E+05	1.42E+05
6.09E+00	8.05E+04	8.60E+04	8.91E+04	8.07E+04	8.64E+04	8.95E+04
2.19E+01	4.61E+04	5.02E+04	5.23E+04	4.61E+04	5.04E+04	5.26E+04
7.87E+01	2.32E+04	2.71E+04	2.90E+04	2.32E+04	2.73E+04	2.92E+04
2.83E+02	8.41E+03	1.16E+04	1.32E+04	8.45E+03	1.17E+04	1.33E+04
1.02E+03	2.16E+03	3.57E+03	4.35E+03	2.19E+03	3.63E+03	4.42E+03
3.65E+03	7.02E+02	1.29E+03	1.63E+03	7.09E+02	1.31E+03	1.67E+03
5.48E+03	6.13E+02	1.11E+03	1.40E+03	6.19E+02	1.14E+03	1.43E+03
6.73E+03	5.70E+02	1.04E+03	1.30E+03	5.76E+02	1.06E+03	1.33E+03
8.26E+03	5.25E+02	9.53E+02	1.19E+03	5.31E+02	9.73E+02	1.22E+03
1.01E+04	4.77E+02	8.66E+02	1.08E+03	4.83E+02	8.86E+02	1.11E+03
1.24E+04	4.25E+02	7.73E+02	9.61E+02	4.32E+02	7.92E+02	9.89E+02
1.53E+04	3.71E+02	6.77E+02	8.38E+02	3.78E+02	6.95E+02	8.65E+02
1.88E+04	3.16E+02	5.79E+02	7.15E+02	3.23E+02	5.97E+02	7.40E+02
2.30E+04	2.63E+02	4.84E+02	5.96E+02	2.70E+02	5.01E+02	6.20E+02
2.83E+04	2.14E+02	3.96E+02	4.86E+02	2.20E+02	4.13E+02	5.09E+02
3.47E+04	1.71E+02	3.20E+02	3.90E+02	1.77E+02	3.35E+02	4.12E+02
3.65E+04	1.61E+02	3.03E+02	3.69E+02	1.68E+02	3.18E+02	3.90E+02

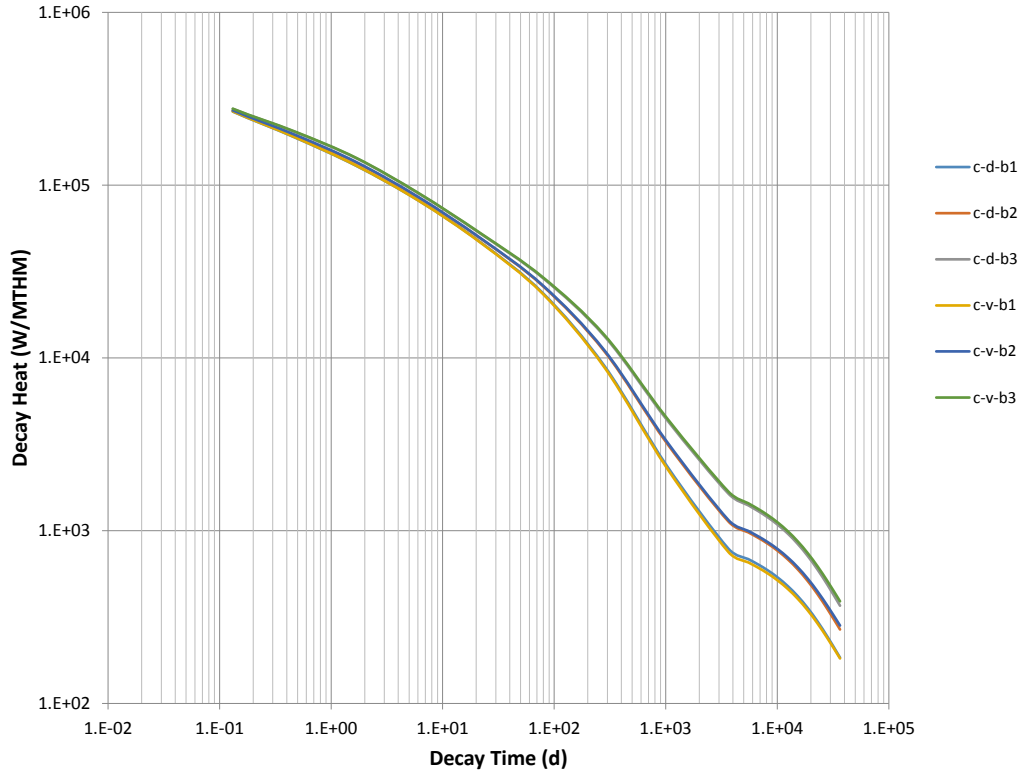


Figure 23. Comparison of “C” assembly decay heat trajectories in the BFN equilibrium LEU core – 100 years.

Table 18. Decay heat (W/MTHM) for the fuel “C” assembly in the BFN LEU core

Time (d)	Dominant zone			Vanished zone		
	b1	b2	b3	b1	b2	b3
0.00E+00	1.80E+06	1.77E+06	1.75E+06	1.80E+06	1.77E+06	1.75E+06
3.65E-02	3.93E+05	3.93E+05	3.97E+05	3.94E+05	3.94E+05	3.98E+05
1.31E-01	2.67E+05	2.70E+05	2.76E+05	2.68E+05	2.71E+05	2.77E+05
4.72E-01	1.89E+05	1.95E+05	2.03E+05	1.90E+05	1.96E+05	2.05E+05
1.70E+00	1.29E+05	1.35E+05	1.43E+05	1.29E+05	1.36E+05	1.44E+05
6.09E+00	8.15E+04	8.47E+04	8.94E+04	8.13E+04	8.52E+04	9.00E+04
2.19E+01	4.67E+04	4.91E+04	5.23E+04	4.64E+04	4.93E+04	5.27E+04
7.87E+01	2.38E+04	2.62E+04	2.91E+04	2.36E+04	2.64E+04	2.94E+04
2.83E+02	8.98E+03	1.09E+04	1.33E+04	8.80E+03	1.10E+04	1.36E+04
1.02E+03	2.39E+03	3.23E+03	4.41E+03	2.32E+03	3.30E+03	4.51E+03
3.65E+03	7.87E+02	1.13E+03	1.64E+03	7.52E+02	1.15E+03	1.68E+03
5.48E+03	6.86E+02	9.80E+02	1.41E+03	6.56E+02	9.98E+02	1.44E+03
6.73E+03	6.39E+02	9.12E+02	1.30E+03	6.11E+02	9.29E+02	1.34E+03
8.26E+03	5.89E+02	8.40E+02	1.19E+03	5.64E+02	8.57E+02	1.23E+03
1.01E+04	5.35E+02	7.64E+02	1.08E+03	5.14E+02	7.80E+02	1.11E+03
1.24E+04	4.78E+02	6.83E+02	9.62E+02	4.60E+02	6.99E+02	9.92E+02
1.53E+04	4.18E+02	5.98E+02	8.38E+02	4.03E+02	6.14E+02	8.67E+02
1.88E+04	3.57E+02	5.12E+02	7.15E+02	3.46E+02	5.28E+02	7.41E+02
2.30E+04	2.98E+02	4.28E+02	5.95E+02	2.90E+02	4.44E+02	6.20E+02
2.83E+04	2.43E+02	3.51E+02	4.85E+02	2.38E+02	3.66E+02	5.09E+02
3.47E+04	1.95E+02	2.83E+02	3.90E+02	1.92E+02	2.98E+02	4.12E+02
3.65E+04	1.84E+02	2.68E+02	3.69E+02	1.82E+02	2.83E+02	3.90E+02



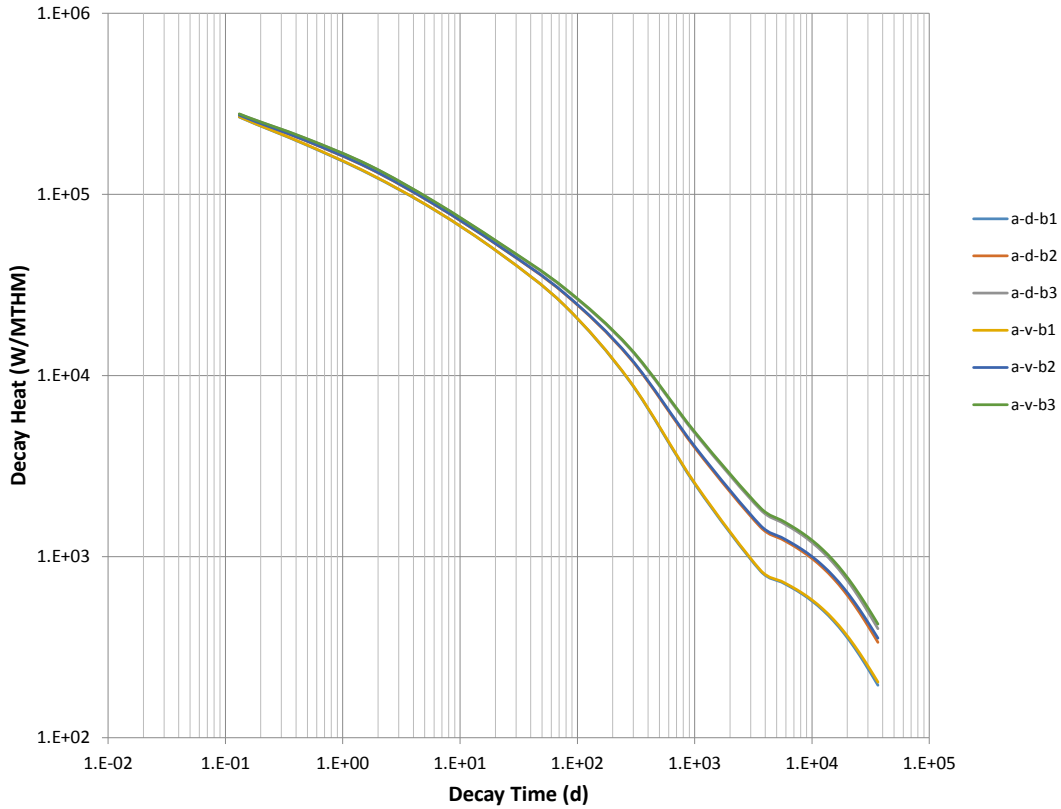


Figure 24. Comparison of “A” assembly decay heat trajectories in the BFN equilibrium MOX core – 100 years.

Table 19. Decay heat (W/MTHM) for the fuel “A” assembly in the BFN MOX core

Time (d)	Dominant zone			Vanished zone		
	b1	b2	b3	b1	b2	b3
0.00E+00	1.80E+06	1.76E+06	1.75E+06	1.80E+06	1.76E+06	1.76E+06
3.65E-02	3.93E+05	3.94E+05	3.97E+05	3.94E+05	3.96E+05	3.99E+05
1.31E-01	2.67E+05	2.73E+05	2.77E+05	2.68E+05	2.74E+05	2.78E+05
4.72E-01	1.89E+05	1.99E+05	2.05E+05	1.90E+05	2.00E+05	2.06E+05
1.70E+00	1.29E+05	1.39E+05	1.44E+05	1.30E+05	1.39E+05	1.45E+05
6.09E+00	8.18E+04	8.75E+04	9.06E+04	8.21E+04	8.79E+04	9.11E+04
2.19E+01	4.72E+04	5.13E+04	5.34E+04	4.72E+04	5.15E+04	5.37E+04
7.87E+01	2.42E+04	2.81E+04	3.00E+04	2.42E+04	2.83E+04	3.02E+04
2.83E+02	9.19E+03	1.24E+04	1.40E+04	9.25E+03	1.25E+04	1.41E+04
1.02E+03	2.49E+03	3.95E+03	4.76E+03	2.52E+03	4.02E+03	4.84E+03
3.65E+03	8.31E+02	1.45E+03	1.81E+03	8.41E+02	1.49E+03	1.86E+03
5.48E+03	7.24E+02	1.25E+03	1.55E+03	7.33E+02	1.28E+03	1.59E+03
6.73E+03	6.74E+02	1.16E+03	1.44E+03	6.82E+02	1.19E+03	1.47E+03
8.26E+03	6.21E+02	1.07E+03	1.32E+03	6.30E+02	1.10E+03	1.35E+03
1.01E+04	5.65E+02	9.71E+02	1.19E+03	5.73E+02	9.96E+02	1.23E+03
1.24E+04	5.05E+02	8.66E+02	1.06E+03	5.13E+02	8.90E+02	1.09E+03
1.53E+04	4.41E+02	7.57E+02	9.21E+02	4.50E+02	7.80E+02	9.52E+02
1.88E+04	3.77E+02	6.47E+02	7.83E+02	3.86E+02	6.69E+02	8.13E+02
2.30E+04	3.15E+02	5.41E+02	6.51E+02	3.23E+02	5.61E+02	6.79E+02
2.83E+04	2.57E+02	4.42E+02	5.30E+02	2.65E+02	4.62E+02	5.56E+02
3.47E+04	2.06E+02	3.56E+02	4.24E+02	2.14E+02	3.75E+02	4.49E+02
3.65E+04	1.95E+02	3.37E+02	4.01E+02	2.03E+02	3.55E+02	4.25E+02

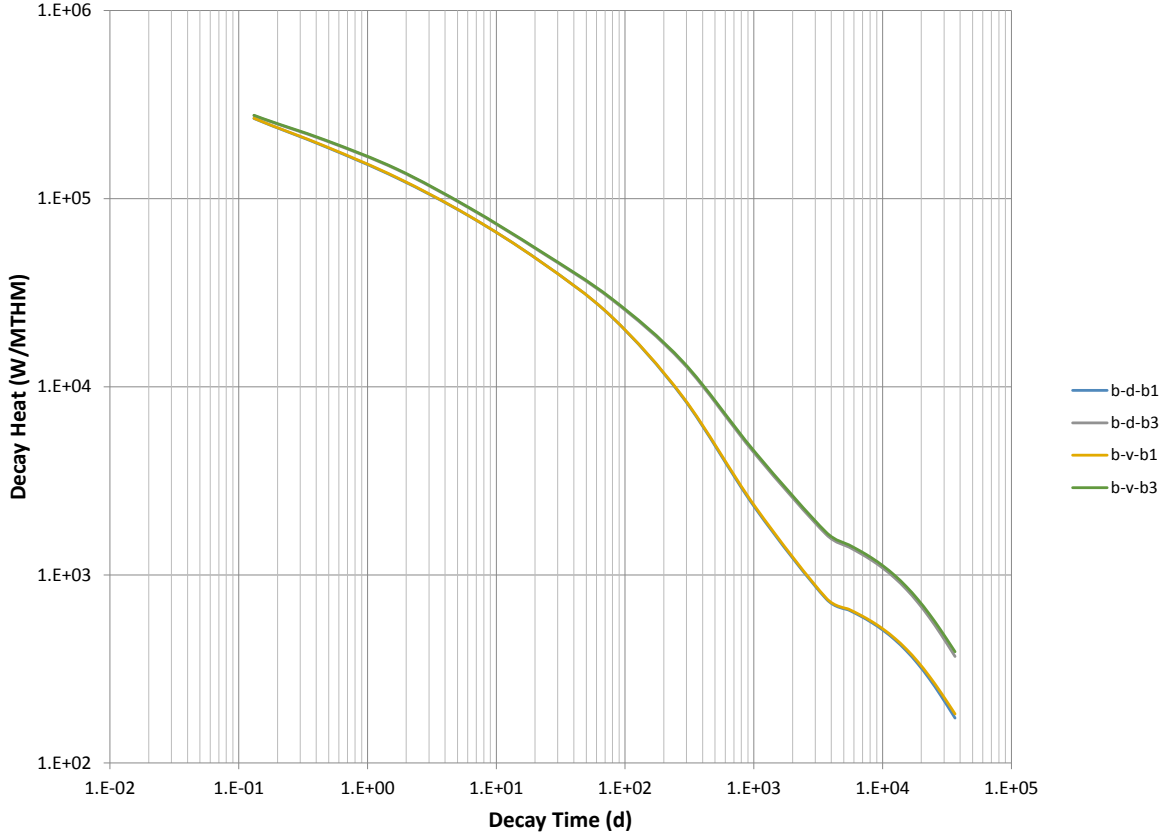


Figure 25. Comparison of “B” assembly decay heat trajectories in the BFN equilibrium MOX core – 100 years.

Table 20. Decay heat (W/MTHM) for the fuel “B” assembly in the BFN MOX core

Time (d)	Dominant Zone			Vanished Zone		
	b1	b2	b3	b1	b2	b3
0.00E+00	1.80E+06		1.75E+06	1.80E+06		1.75E+06
3.65E-02	3.93E+05		3.97E+05	3.94E+05		3.98E+05
1.31E-01	2.67E+05		2.76E+05	2.68E+05		2.77E+05
4.72E-01	1.89E+05		2.03E+05	1.90E+05		2.5E+05
1.70E+00	1.29E+05		1.43E+05	1.29E+05		1.44E+05
6.09E+00	8.10E+04		8.94E+04	8.13E+04		9.00E+04
2.19E+01	4.64E+04		5.23E+04	4.64E+04		5.27E+04
7.87E+01	2.35E+04		2.91E+04	2.36E+04		2.94E+04
2.83E+02	8.71E+03		1.33E+04	8.80E+03		1.36E+04
1.02E+03	2.28E+03		4.41E+03	2.32E+03		4.51E+03
3.65E+03	7.44E+02		1.64E+03	7.52E+02		1.68E+03
5.48E+03	6.49E+02		1.41E+03	6.56E+02		1.44E+03
6.73E+03	6.04E+02		1.30E+03	6.11E+02		1.34E+03
8.26E+03	5.57E+02		1.19E+03	5.64E+02		1.23E+03
1.01E+04	5.06E+02		1.08E+03	5.14E+02		1.11E+03
1.24E+04	4.52E+02		9.62E+02	4.60E+02		9.92E+02
1.53E+04	3.95E+02		8.38E+02	4.03E+02		8.67E+02
1.88E+04	3.37E+02		7.15E+02	3.46E+02		7.41E+02
2.30E+04	2.81E+02		5.95E+02	2.90E+02		6.20E+02
2.83E+04	2.29E+02		4.85E+02	2.38E+02		5.09E+02
3.47E+04	1.83E+02		3.90E+02	1.92E+02		4.12E+02
3.65E+04	1.73E+02		3.69E+02	1.82E+02		3.90E+02

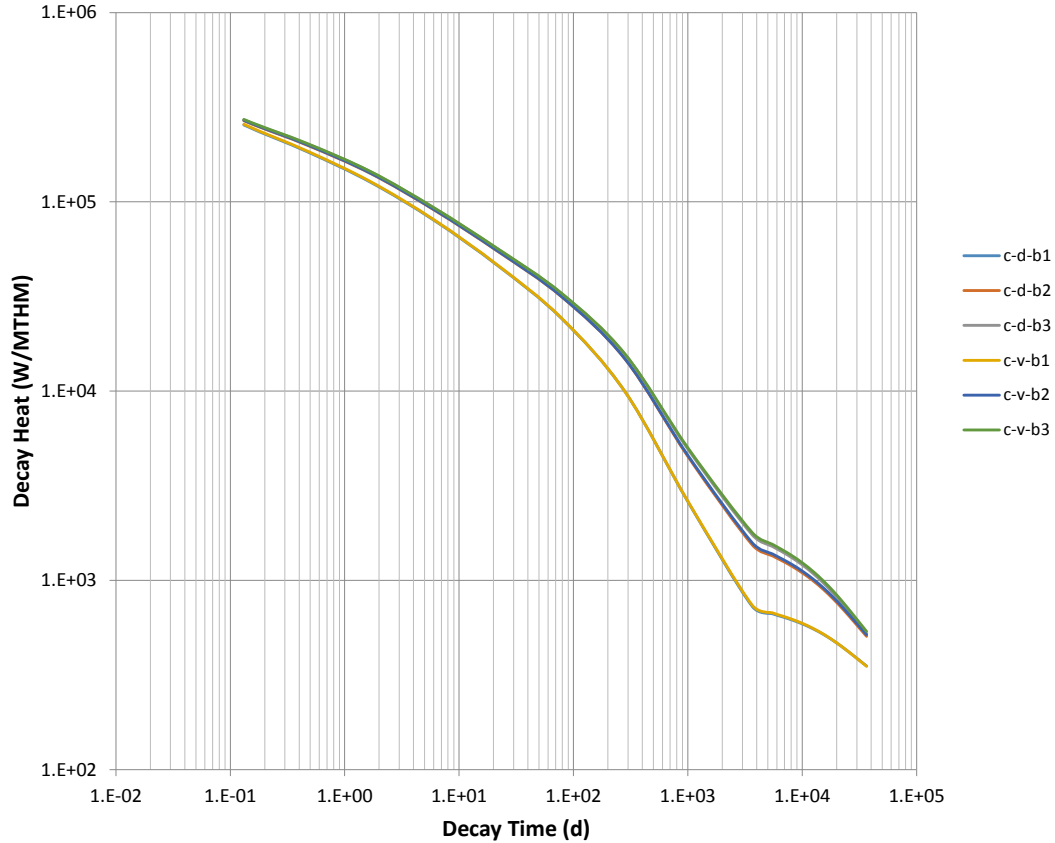


Figure 26. Comparison of “C” assembly decay heat trajectories in the BFN equilibrium MOX core – 100 years.

Table 21. Decay heat (W/MTHM) for the fuel “C” assembly in the BFN MOX core

Time (d)	Dominant zone			Vanished zone		
	b1	b2	b3	b1	b2	b3
0.00E+00	1.69E+06	1.71E+06	1.71E+06	1.70E+06	1.72E+06	1.72E+06
3.65E-02	3.73E+05	3.84E+05	3.87E+05	3.75E+05	3.86E+05	3.89E+05
1.31E-01	2.55E+05	2.68E+05	2.71E+05	2.57E+05	2.70E+05	2.73E+05
4.72E-01	1.84E+05	1.99E+05	2.02E+05	1.85E+05	2.00E+05	2.03E+05
1.70E+00	1.27E+05	1.41E+05	1.44E+05	1.28E+05	1.42E+05	1.44E+05
6.09E+00	7.96E+04	9.03E+04	9.23E+04	8.01E+04	9.08E+04	9.27E+04
2.19E+01	4.59E+04	5.46E+04	5.61E+04	4.61E+04	5.49E+04	5.64E+04
7.87E+01	2.42E+04	3.15E+04	3.27E+04	2.43E+04	3.16E+04	3.29E+04
2.83E+02	9.90E+03	1.47E+04	1.56E+04	9.91E+03	1.48E+04	1.57E+04
1.02E+03	2.56E+03	4.45E+03	4.87E+03	2.58E+03	4.52E+03	4.94E+03
3.65E+03	7.29E+02	1.54E+03	1.74E+03	7.40E+02	1.58E+03	1.79E+03
5.48E+03	6.66E+02	1.35E+03	1.51E+03	6.74E+02	1.38E+03	1.55E+03
6.73E+03	6.41E+02	1.27E+03	1.41E+03	6.48E+02	1.30E+03	1.45E+03
8.26E+03	6.15E+02	1.18E+03	1.31E+03	6.21E+02	1.21E+03	1.34E+03
1.01E+04	5.87E+02	1.09E+03	1.20E+03	5.91E+02	1.12E+03	1.23E+03
1.24E+04	5.54E+02	9.95E+02	1.09E+03	5.58E+02	1.02E+03	1.11E+03
1.53E+04	5.18E+02	8.96E+02	9.70E+02	5.21E+02	9.15E+02	9.94E+02
1.88E+04	4.79E+02	7.95E+02	8.52E+02	4.81E+02	8.12E+02	8.74E+02
2.30E+04	4.39E+02	6.97E+02	7.39E+02	4.40E+02	7.12E+02	7.58E+02
2.83E+04	3.99E+02	6.06E+02	6.35E+02	3.99E+02	6.18E+02	6.51E+02
3.47E+04	3.61E+02	5.24E+02	5.43E+02	3.61E+02	5.35E+02	5.58E+02
3.65E+04	3.52E+02	5.06E+02	5.23E+02	3.52E+02	5.17E+02	5.37E+02

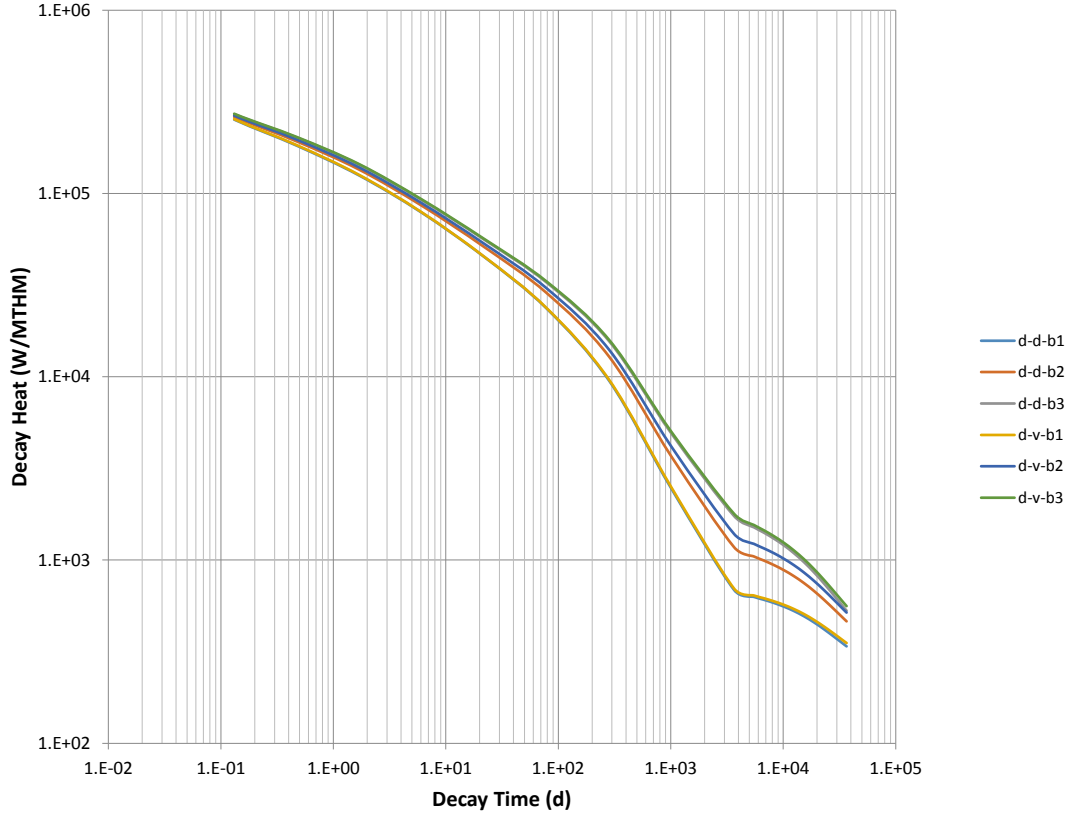


Figure 27. Comparison of “D” assembly decay heat trajectories in the BFN equilibrium MOX core – 100 years.

Table 22. Decay heat (W/MTHM) for the fuel “D” assembly in the BFN MOX core

Time (d)	Dominant zone			Vanished zone		
	b1	b2	b3	b1	b2	b3
0.00E+00	1.68E+06	1.69E+06	1.71E+06	1.68E+06	1.71E+06	1.72E+06
3.65E-02	3.71E+05	3.78E+05	3.87E+05	3.72E+05	3.82E+05	3.89E+05
1.31E-01	2.54E+05	2.62E+05	2.71E+05	2.54E+05	2.66E+05	2.72E+05
4.72E-01	1.83E+05	1.93E+05	2.02E+05	1.84E+05	1.97E+05	2.03E+05
1.70E+00	1.26E+05	1.35E+05	1.44E+05	1.26E+05	1.39E+05	1.45E+05
6.09E+00	7.87E+04	8.59E+04	9.23E+04	7.89E+04	8.86E+04	9.29E+04
2.19E+01	4.51E+04	5.11E+04	5.60E+04	4.52E+04	5.32E+04	5.65E+04
7.87E+01	2.36E+04	2.87E+04	3.27E+04	2.37E+04	3.04E+04	3.31E+04
2.83E+02	9.52E+03	1.29E+04	1.57E+04	9.62E+03	1.40E+04	1.59E+04
1.02E+03	2.44E+03	3.66E+03	4.89E+03	2.48E+03	4.14E+03	4.99E+03
3.65E+03	6.86E+02	1.18E+03	1.74E+03	6.98E+02	1.39E+03	1.79E+03
5.48E+03	6.29E+02	1.05E+03	1.51E+03	6.40E+02	1.23E+03	1.55E+03
6.73E+03	6.06E+02	9.94E+02	1.41E+03	6.18E+02	1.16E+03	1.45E+03
8.26E+03	5.83E+02	9.40E+02	1.31E+03	5.95E+02	1.09E+03	1.35E+03
1.01E+04	5.56E+02	8.81E+02	1.20E+03	5.69E+02	1.02E+03	1.24E+03
1.24E+04	5.27E+02	8.17E+02	1.09E+03	5.40E+02	9.37E+02	1.13E+03
1.53E+04	4.93E+02	7.49E+02	9.74E+02	5.07E+02	8.54E+02	1.01E+03
1.88E+04	4.57E+02	6.79E+02	8.58E+02	4.71E+02	7.69E+02	8.93E+02
2.30E+04	4.19E+02	6.08E+02	7.45E+02	4.33E+02	6.85E+02	7.79E+02
2.83E+04	3.81E+02	5.40E+02	6.41E+02	3.96E+02	6.05E+02	6.74E+02
3.47E+04	3.46E+02	4.78E+02	5.50E+02	3.61E+02	5.33E+02	5.82E+02
3.65E+04	3.38E+02	4.64E+02	5.30E+02	3.52E+02	5.17E+02	5.61E+02

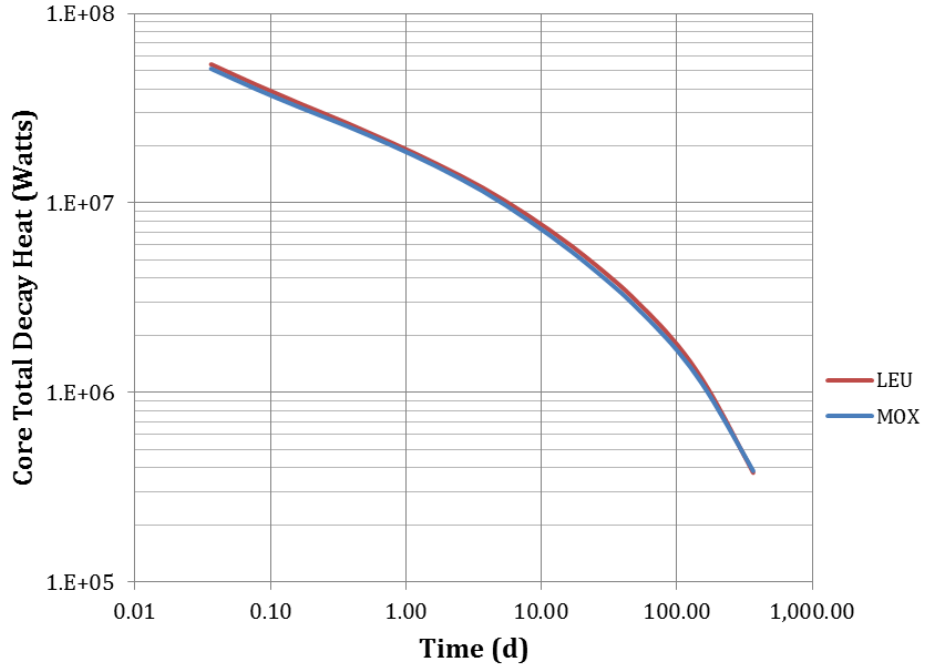


Figure 28. Comparison of decay heat in BFN equilibrium MOX and LEU cores – 1 year.

Table 23. Decay heat for the BFN equilibrium cores – 1 year

Time (d)	LEU (W)	MOX (W)	Time (d)	LEU (W)	MOX (W)
0.00E+00	2.57E+08	2.40E+08	5.48E+01	2.86E+06	2.64E+06
3.65E-03	5.40E+07	5.12E+07	6.73E+01	2.46E+06	2.28E+06
1.02E-02	4.21E+07	3.99E+07	8.26E+01	2.11E+06	1.95E+06
2.83E-02	3.32E+07	3.17E+07	1.01E+02	1.77E+06	1.65E+06
7.87E-02	2.65E+07	2.55E+07	1.24E+02	1.47E+06	1.37E+06
2.19E-01	2.08E+07	2.02E+07	1.53E+02	1.18E+06	1.12E+06
6.09E-01	1.61E+07	1.56E+07	1.88E+02	9.25E+05	8.85E+05
1.70E+00	1.21E+07	1.16E+07	2.30E+02	7.08E+05	6.89E+05
4.72E+00	8.64E+06	8.17E+06	2.83E+02	5.35E+05	5.32E+05
1.31E+01	5.89E+06	5.50E+06	3.47E+02	4.04E+05	4.11E+05
3.65E+01	3.76E+06	3.49E+06	3.65E+02	3.77E+05	3.86E+05

## 10. SUMMARY

This study has calculated

- MOX and LEU core average nuclide concentrations and core nuclide loads,
- MOX core to LEU core nuclide ratios,
- radioactivity sources, and
- decay heat curves for individual fuel assemblies and core-average decay heat values

for Sequoyah (PWR) and Browns Ferry (BWR) reactors. The SCALE/TRITON models utilized the mechanical design of the actual SQN and BFN assemblies with current proposed core (Pu-Am) assay and U-235 enrichment loadings (LEU and MOX) at SQN and BFN. The transport models used for the depletion calculations are accurate 2D representations of the proposed assembly lattices including all fuel rod assays/enrichments and locations, geometrical heterogeneities, and cross-section adjustments for individually calculated Dancoff factors in the BWR lattices. The Dancoff factors and the depletion models for the BFN cores used the average void concentration in the dominant and vanished regions of the ATRIUM 10 lattices. The fuel rod Dancoff factors automatically calculated by TRITON were deemed to be of acceptable accuracy for the SQN assemblies and were used as such in the depletion models for the SQN cores.

The SQN lattices were assumed to cover the full axial fuel length. There are no adjustments for grid spacer regions or axial blankets.

The BFN assemblies were assumed to consist of a lower dominant lattice and an upper vanished lattice, where part length rods had disappeared. In some cases, either the lower or upper region may have several lattices of slightly varying enrichment. For this study, the major lattice in the lower and upper zones was assumed to extend for the full length of that zone.

Depletion calculations for chosen lattices were carried out to ~60 GWd/MTHM burnup to ensure that the burnup ranges attained within the cores were covered. For a given assembly type in a core, there were two to three burnup “classes” that could be characterized as low, medium, and high. The average burnup in each of these classes was determined for each assembly type based on the burnup distributions from the equilibrium core fuel management studies. The nuclide concentrations (g/MTHM) were interpolated at the corresponding average burnup of a burnup class of a particular fuel assembly type. The core load for each nuclide was calculated as the weighted sum of each nuclide concentration times the number of metric tonnes of heavy metals in the core. This same weighting procedure was applied to the radioactive source strengths and the core decay heat generation.

The results of the MOX/LEU concentration ratios were compared to the previously employed (PWR) ratios [3] in Table 7 (for SQN) and in Table 9 (for BFN) of the current report. For many nuclides, the differences between the newly calculated and the previously reported nuclide ratios are minimal (<1–2%). The differences for the plutonium isotopes (7–22%) and for the curium isotopes (18–42%) are more substantial. These larger differences are the result of the use of more modern, up-to-date computational methods and nuclear data in the current study.

The same modeling methods and nuclear data as used in this study were also used previously to benchmark nuclide measurements in high-burnup BWR assemblies [10]; for those cases, the agreement between the experimental and calculated concentrations for the plutonium isotopes was between 0.7 and 3.4%. The differences noted between the plutonium results documented in the current study and those previously reported [3] are likely due to the use of more modern, up-to-date computational methods and

nuclear data; the current study used the more recent ENDF-B/VII cross sections as well as the sophisticated cross section self-shielding and geometry capabilities of the TRITON code system available in 2011, which did not exist in 1995.

The conspicuous difference for the nuclide Mo-92 production in the MOX and LEU cores is a direct result of the much greater fission yield of this nuclide by plutonium isotopes compared to the fission yield of this nuclide by U-235.

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